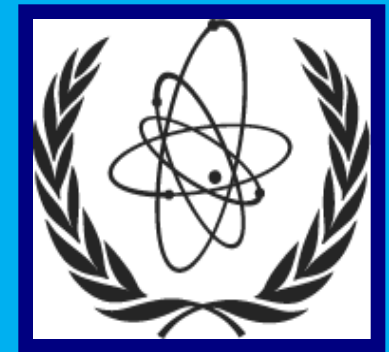


NUCLEAR DATA EVALUATION METHODOLOGY INCLUDING ESTIMATES OF COVARIANCE



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OUTLINE

- ❑ Overview of Nuclear Data Evaluation Methods
- ❑ Selection of experimental data and EXFOR
- ❑ Experimental uncertainties and correlations
- ❑ Modelling uncertainties
 - Model defects
 - Model parameters
- ❑ Evaluation based on GLSQ
- ❑ (UMC formulation)
- ❑ (UMC+TMC)



What is Nuclear (Reaction) Data Evaluation?

A properly weighted combination (usually by GLSQ fit) of selected experimental data (and nuclear reaction modelling results if needed).

Bayesian approaches:

- ❑ “Non-model” GLSQ fit: standards
- ❑ Model prior + GLSQ fit



Nuclear Data Evaluation

Evaluated cross sections and covariance matrices

Experimental Input

Inter and -intra
experiment
correlations

Experimental
cross sections



Prior Knowledge

Model Defects

Parameter
Uncertainties

Model cross
sections

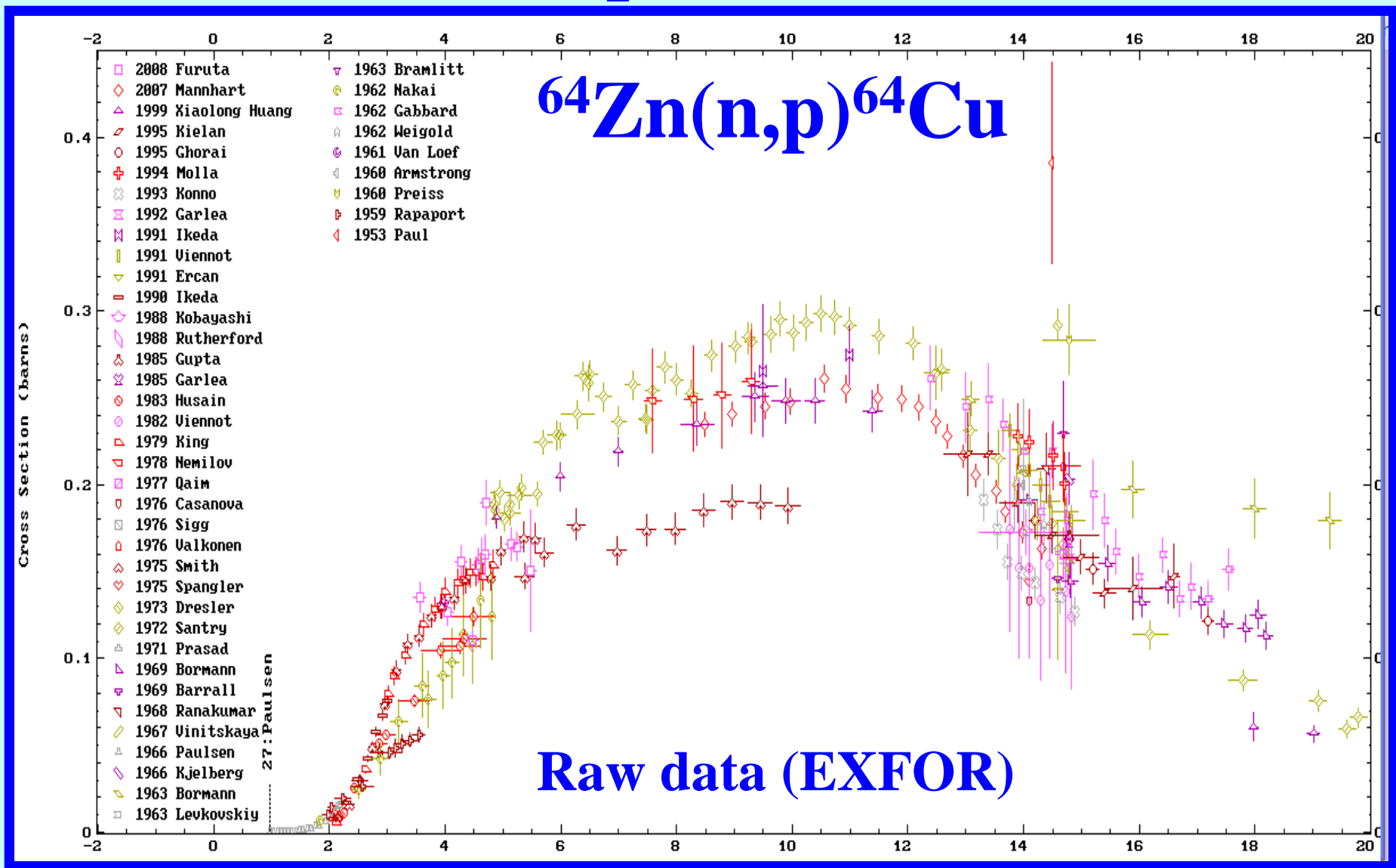
From D. Neudecker, S. Gundacker, H. Leeb *et al.*, ND2010, Jeju Isl., Korea



Experimental uncertainties

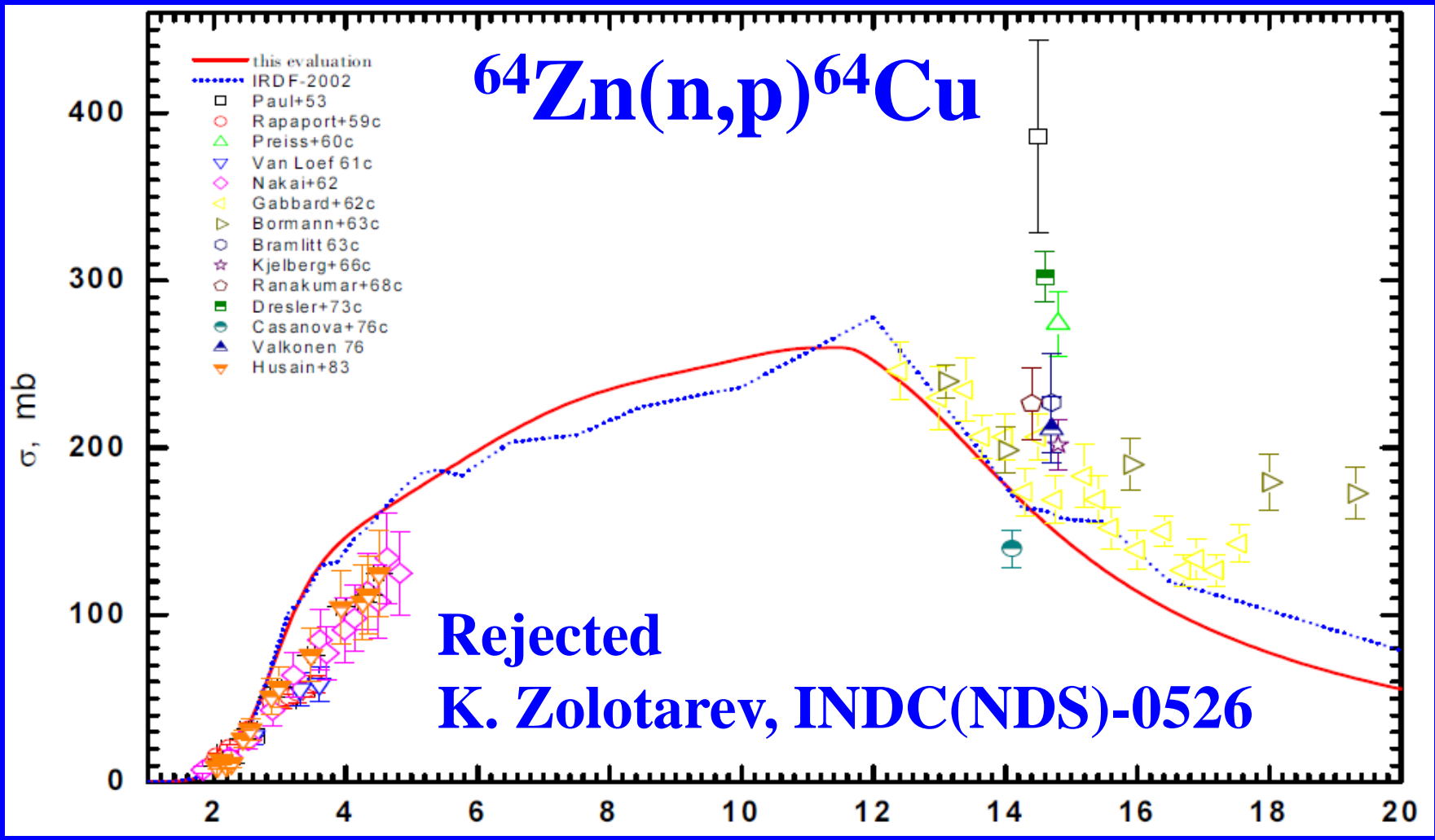


Selection of experimental data (1)



Selection of experimental data (2)

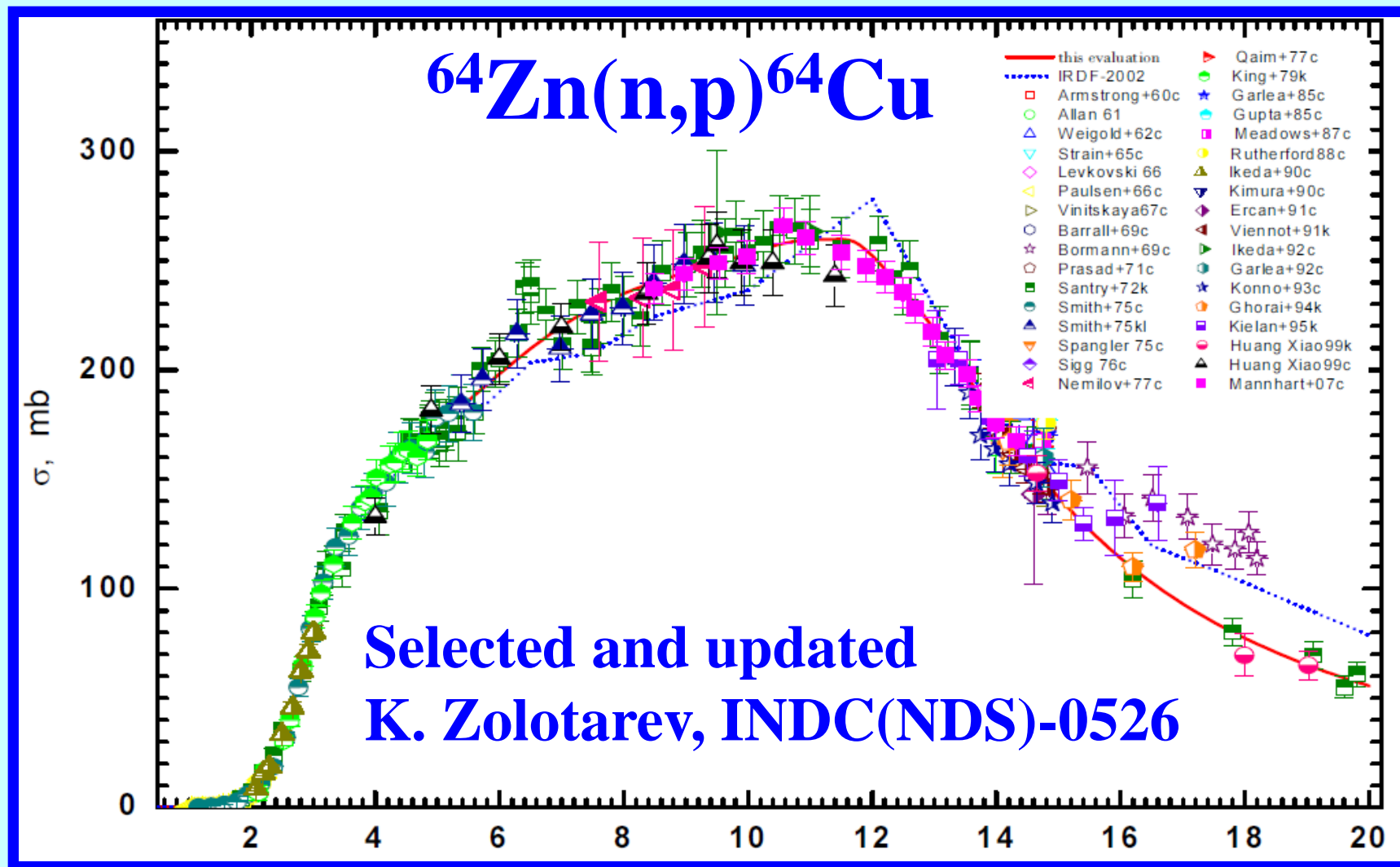
$^{64}\text{Zn}(n,p)^{64}\text{Cu}$



<http://www-nds.iaea.org/reports-new/indc-reports/indc-nds/indc-nds-0526.pdf>



Selection of experimental data (3)



<http://www-nds.iaea.org/reports-new/indc-reports/indc-nds/indc-nds-0526.pdf>



Experimental correlations

“Evaluation of measurement data - Guide to the expression of uncertainty in measurement”

Joint Committee for Guides in Metrology,

JCGM 100:2008, www.bipm.org (2008)

□ Intra experiments correlations:

Short and long term correlations within a single experiment can and should be estimated

(statistical and systematic uncertainty)

□ Inter-experiments correlations

(very often neglected, default zero !!!)



Experimental uncertainties

Energy (MeV)	σ_{Am} (mb)	Unc. (%)	Correlation matrix (x100)										
8.34(15)	96.8	6.5	100										
9.15(15)	162.9	5.7	35	100									
13.33(15)	241.8	4.6	37	42	100								
16.10(15)	152.4	4.6	38	43	53	100							
17.16 (3)	116.1	4.4	40	45	57	58	100						
17.90(10)	105.7	4.4	41	45	57	59	84	100					
19.36(15)	89.5	8.2	21	24	30	31	39	39	100				
19.95 (7)	102.1	5.8	30	34	44	45	58	59	51	100			
20.61 (4)	77.9	8.8	20	22	29	30	40	42	39	65	100		

A. Plompen, ND workshop, ICTP, Trieste 2010



Model uncertainties



Model parameter uncertainties

D.L. Smith, “Covariance Matrices for Nuclear Cross-Sections Derived from Nuclear Model Calculations”.

Report **ANL/NDM-159**, Argonne National Laboratory, 2005

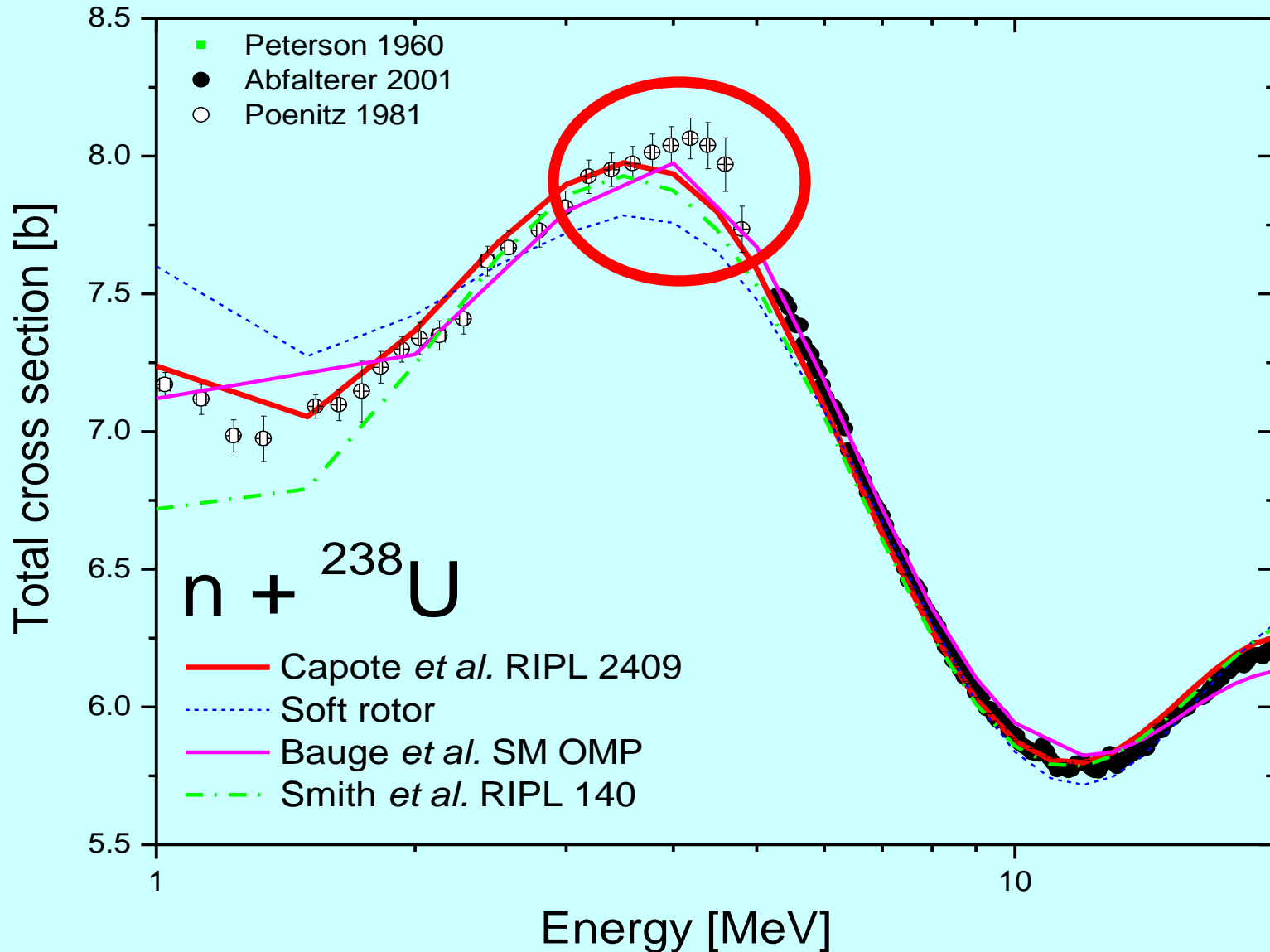
$$\overline{\sigma}_i = \frac{1}{K} \sum_{k=1}^K \sigma_{ik} \quad V_{ij} = \overline{\sigma_i \sigma_j} - \overline{\sigma}_i \times \overline{\sigma}_j$$

i, j - energy indexes

Monte Carlo calculation of covariance first tested by A. Koning



Model defects



EMPIRE code & RIPL database

Extension of the nuclear reaction model code **EMPIRE** to actinides' nuclear data evaluation



@NNDC: <http://www.nndc.bnl.gov/empire219/>

@IAEA: <http://www-nds.iaea.org/empire/>



Available online at www.sciencedirect.com



**Nuclear Data
Sheets**

Nuclear Data Sheets 110 (2009) 3107–3214

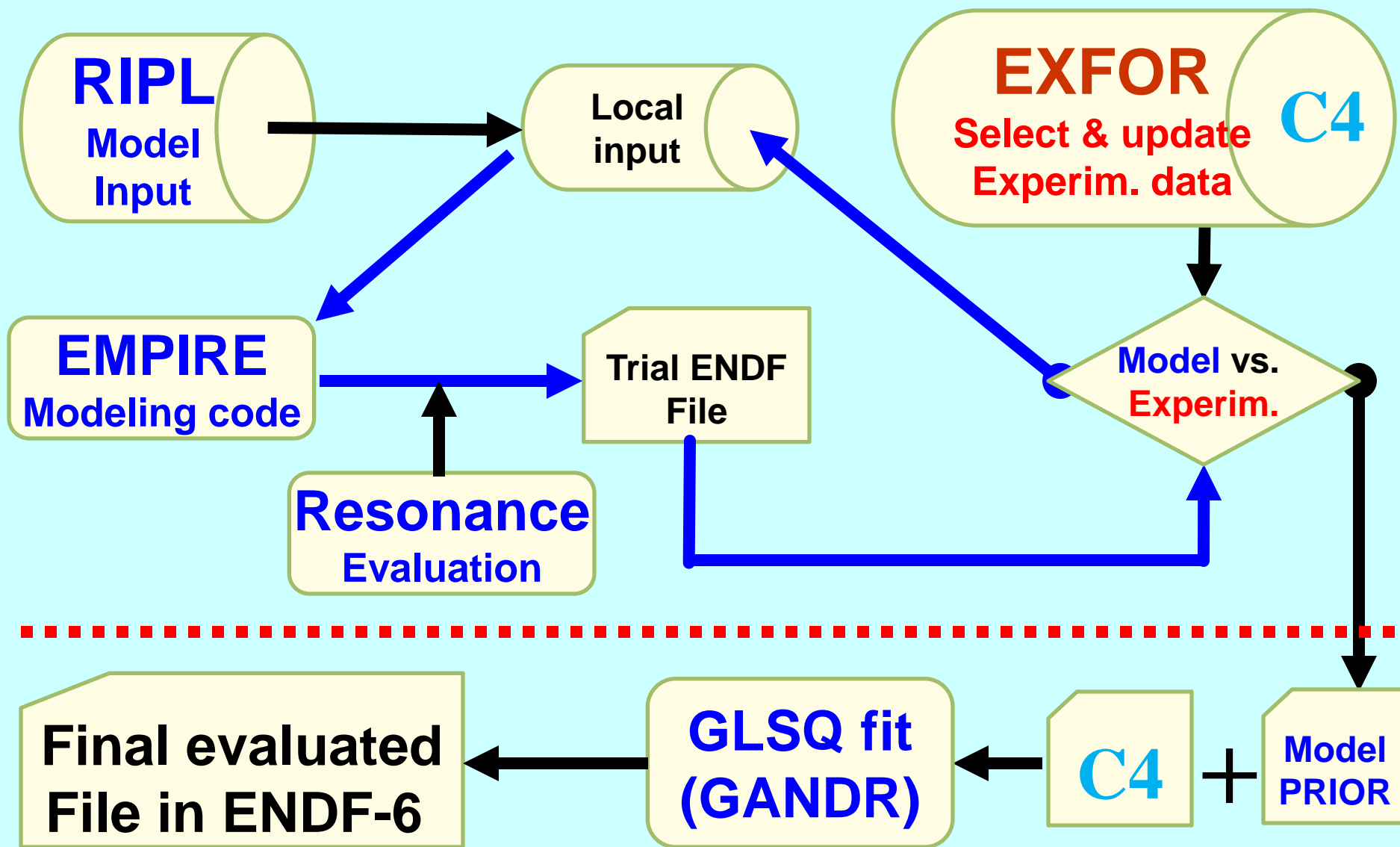
www.elsevier.com/locate/nds

RIPL – Reference Input Parameter Library for Calculation of Nuclear Reactions and Nuclear Data Evaluations

R. Capote,¹ M. Herman,^{1,2} P. Obložinský,^{1,2} P.G. Young,³ S. Goriely,⁴ T. Belgia,⁵ A.V. Ignatyuk,⁶ A.J. Koning,⁷ S. Hilaire,⁸ V.A. Plujko,⁹ M. Avrigeanu,¹⁰ O. Bersillon,⁸ M.B. Chadwick,³ T. Fukahori,¹¹ Zhigang Ge,¹² Yinlu Han,¹² S. Kailas,¹³ J. Kopecky,¹⁴ V.M. Maslov,¹⁵ G. Reffo,¹⁶ M. Sin,¹⁷ E.Sh. Soukhovitskii,¹⁵ and P. Talou³



Nuclear Data Evaluation process



Evaluation – fast energy range

- ❑ Use **nuclear model code** (e.g. **EMPIRE**)
 - ❑ Choose **adequate model** options
 - ❑ Determine **recommended input parameters** (**RIPL**)
 - ❑ Calculate cross sections and other quantities
 - ❑ Compare **calculated values to selected measured data**
(after correcting for new stds, discarding discrepant, etc)
 - ❑ Fine-tune the input model parameters
- Loop-1**
- ❑ From **model parameter uncertainties** and **model uncertainties** generate **covariance matrix prior**

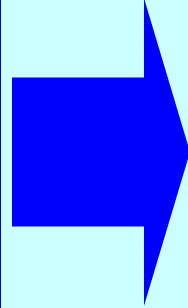


Evaluation methods



Model prior +GLSQ fit

Monte Carlo prior
(model) +
GANDR (GLSQ)

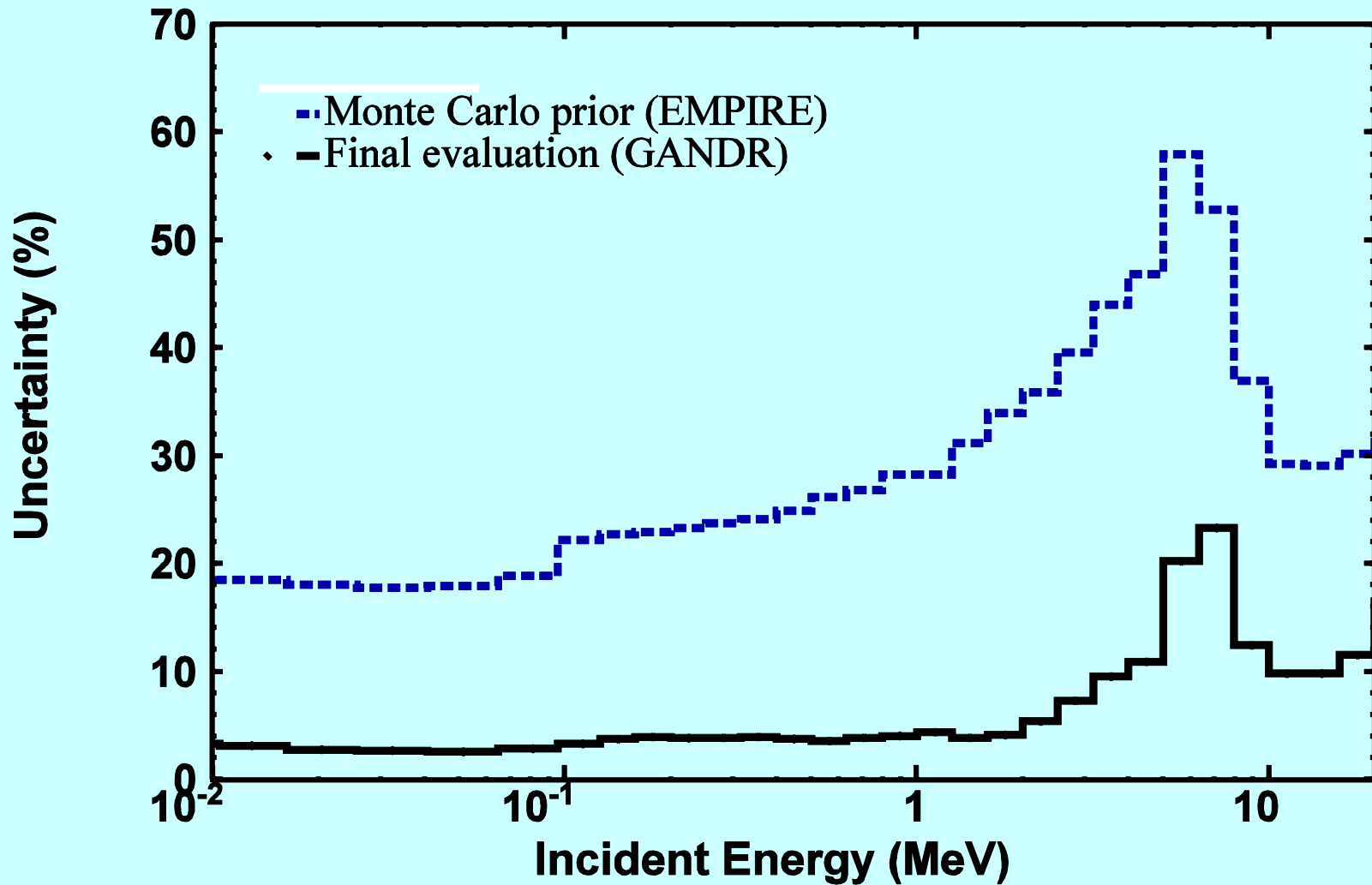


A. Trkov and R. Capote, “Cross-Section Covariance Data”, Th-232 evaluation for ENDF/B-VII.0 (MAT=9040 MF=1 MT=451); Pa-231 and Pa-233 evaluations for ENDF/B-VII.0 (MAT=9133 and 9137 MF=1 MT=451), National Nuclear Data Center, BNL (<http://www.nndc.bnl.gov>), 15 December 2006.

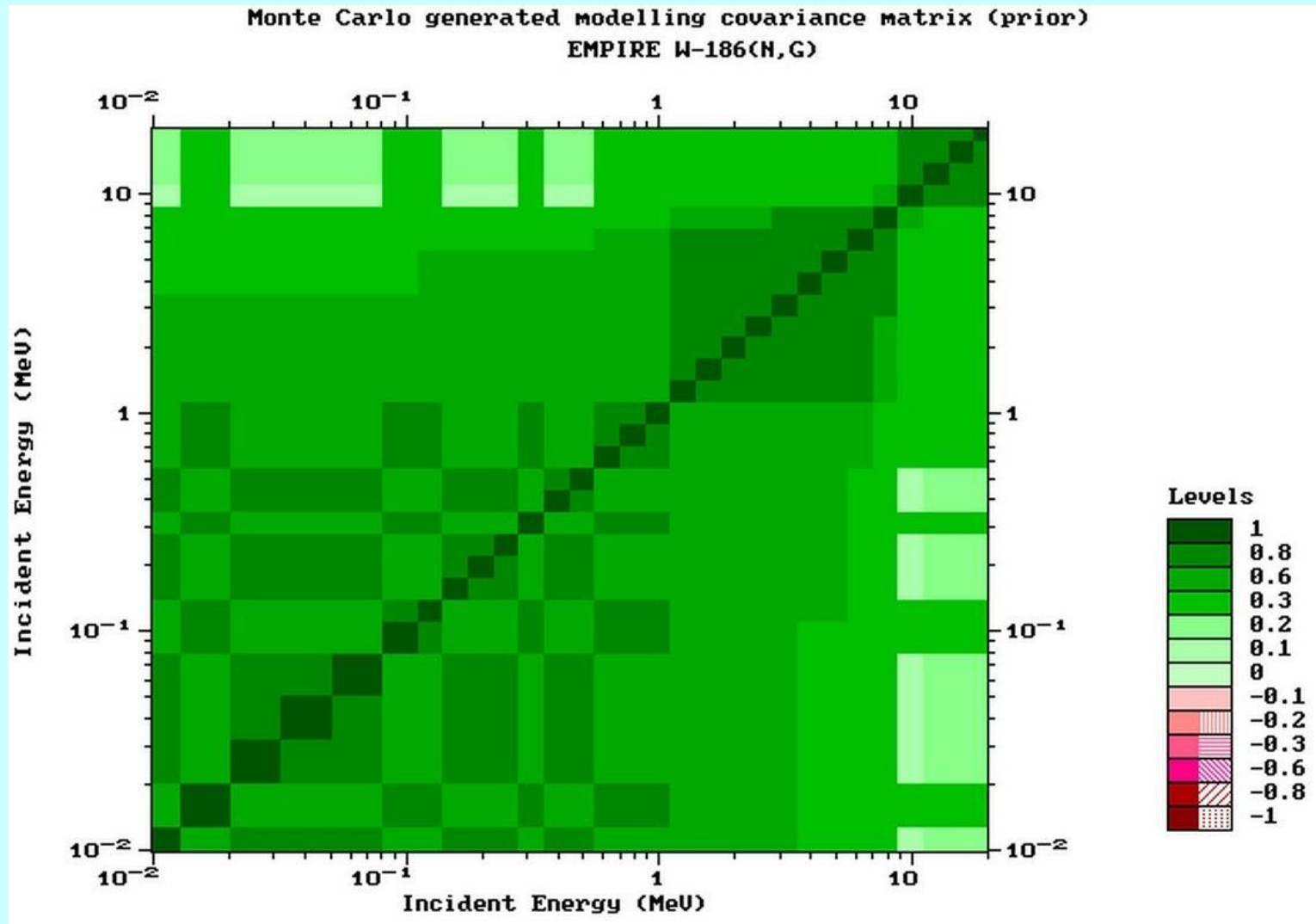
D.W. Muir, **GANDR** project (IAEA),
Online at www-nds.iaea.org/gandr/.



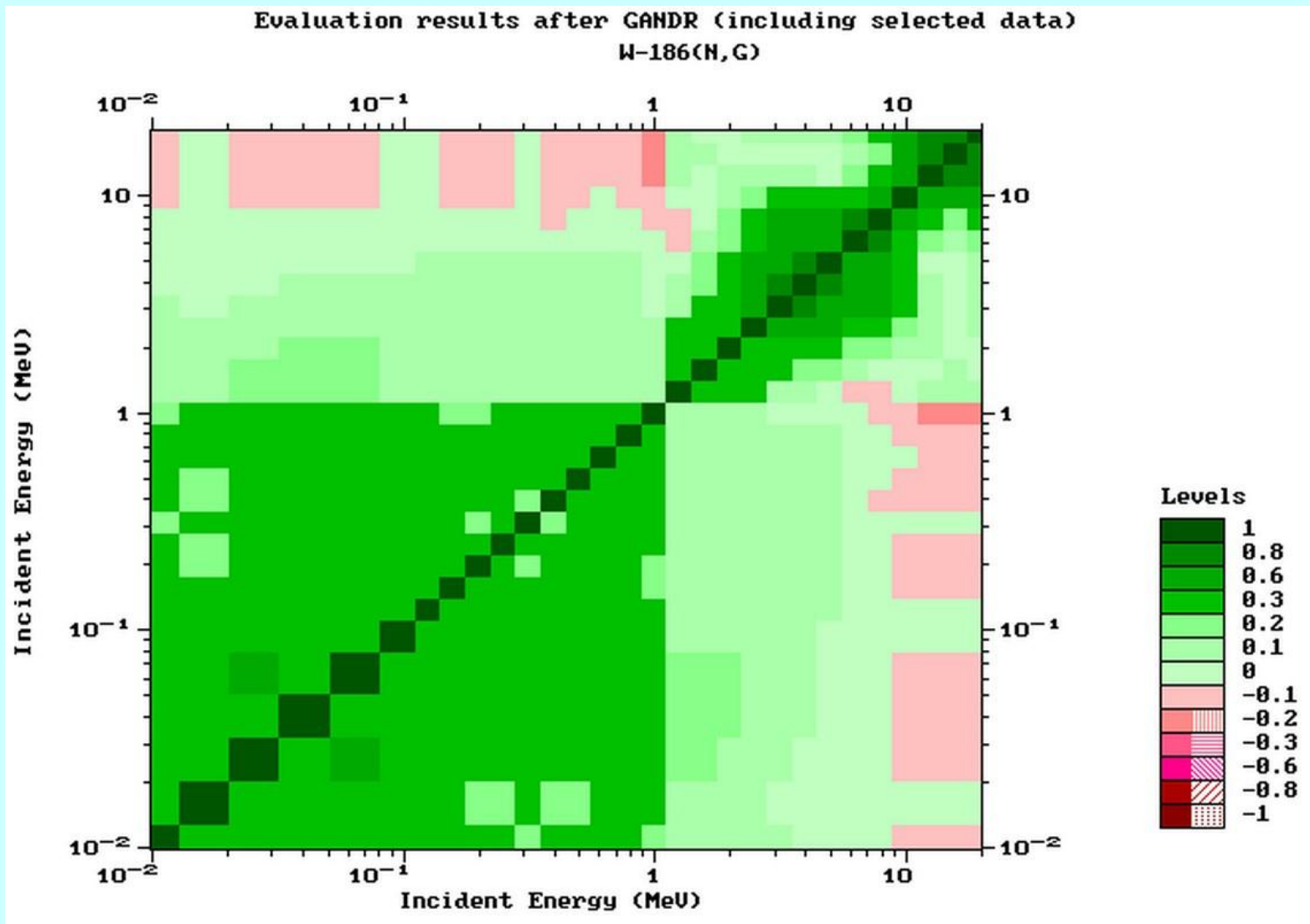
Uncertainties – model vs evaluated



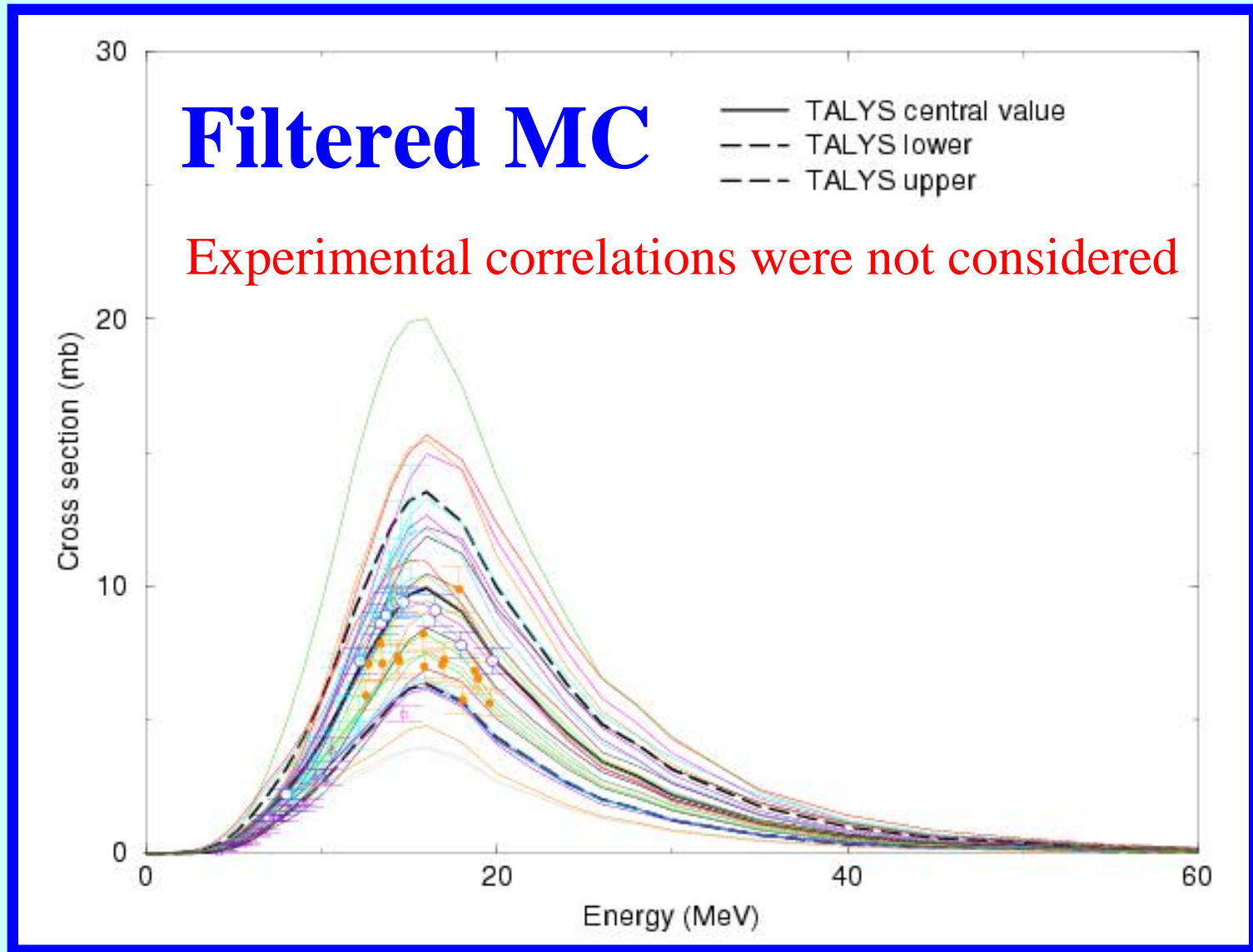
Model prior correlations



Final evaluation correlations



Koning & Rochman approach: TENDL



Take home message

Evaluation: A properly weighted combination (usually by GLSQ fit) of selected experimental data (and nuclear reaction modelling results).

Bayesian approaches

- ❑ “Non-model” GLSQ fit
(e.g. standards & monitor reactions)
- ❑ Model prior + GLSQ fit (incomplete exp. database)
- ❑ FMC (TENDL)

Experimental data and uncertainty analysis



Unified Monte Carlo (UMC)

D.L. Smith, “A Unified Monte Carlo Approach to Fast Neutron Cross Section Data Evaluation,” *Proceedings of the 8th International Topical Meeting on Nuclear Applications and Utilization of Accelerators*, Pocatello, July 29 – August 2, 2007, p. 736.

BAYES THEOREM & PRINCIPLE OF MAXIMUM ENTROPY

$$p(\boldsymbol{\sigma}) = C \times \mathcal{L}(\mathbf{y}_E, \mathbf{V}_E \mid \boldsymbol{\sigma}) \times p_0(\boldsymbol{\sigma} \mid \boldsymbol{\sigma}_C, \mathbf{V}_C)$$

$$p_0(\boldsymbol{\sigma} \mid \boldsymbol{\sigma}_C, \mathbf{V}_C) \sim \exp\{-(1/2)[(\boldsymbol{\sigma}-\boldsymbol{\sigma}_C)^T \cdot (\mathbf{V}_C)^{-1} \cdot (\boldsymbol{\sigma}-\boldsymbol{\sigma}_C)]\}$$

$$\mathcal{L}(\mathbf{y}_E, \mathbf{V}_E \mid \boldsymbol{\sigma}) \sim \exp\{-(1/2)[(\mathbf{y}-\mathbf{y}_E)^T \cdot (\mathbf{V}_E)^{-1} \cdot (\mathbf{y}-\mathbf{y}_E)]\}, \mathbf{y}=f(\boldsymbol{\sigma})$$

$\mathbf{y}_E, \mathbf{V}_E$: measured quantities with “n” elements

$\mathbf{y}_C, \mathbf{V}_C$: calculated using nuclear models with “m” elements

UMC based on $p(\boldsymbol{\sigma})$, GLS on the peak of the distribution



Total Monte Carlo (TMC)

- ❑ Propagating covariance data is an approximation of true uncertainty propagation (especially regarding ENDF-6 format limitations)
- ❑ Covariance data requires extra processing and “satellite software” for application codes
- ❑ Alternative: Create an ENDF-6 file for each random sample and finish the entire physics-to-application loop.

Koning and Rochman, *Ann Nuc En* **35**, 2024 (2008)

Experimental data was included via Filtered MC



TMC+UMC

- ❑ MC model uncertainties (param + model defects)
- ❑ Experimental uncertainties and correlations
- ❑ Uses **UMC (instead of FMC)** to produce samples according to the combined “a posteriori” PDF
- ❑ Create an ENDF-6 file for each random sample and finish the entire physics-to-application loop (**TMC**).

Koning and Rochman, *Ann Nuc En* **35**, 2024 (2008)

UMC could be also used to produce covariance matrix to be stored in ENDF format (approximate treatment)

