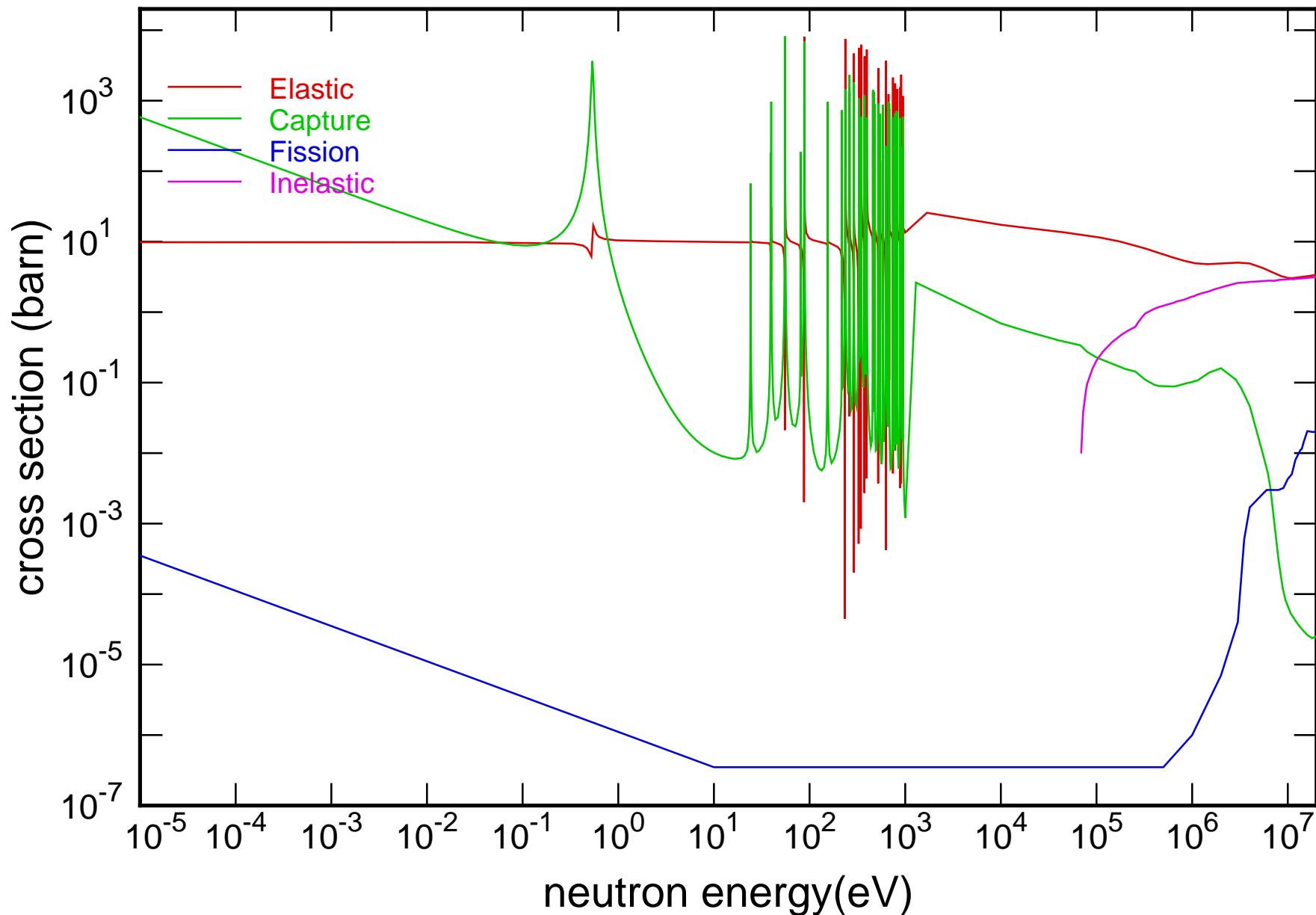
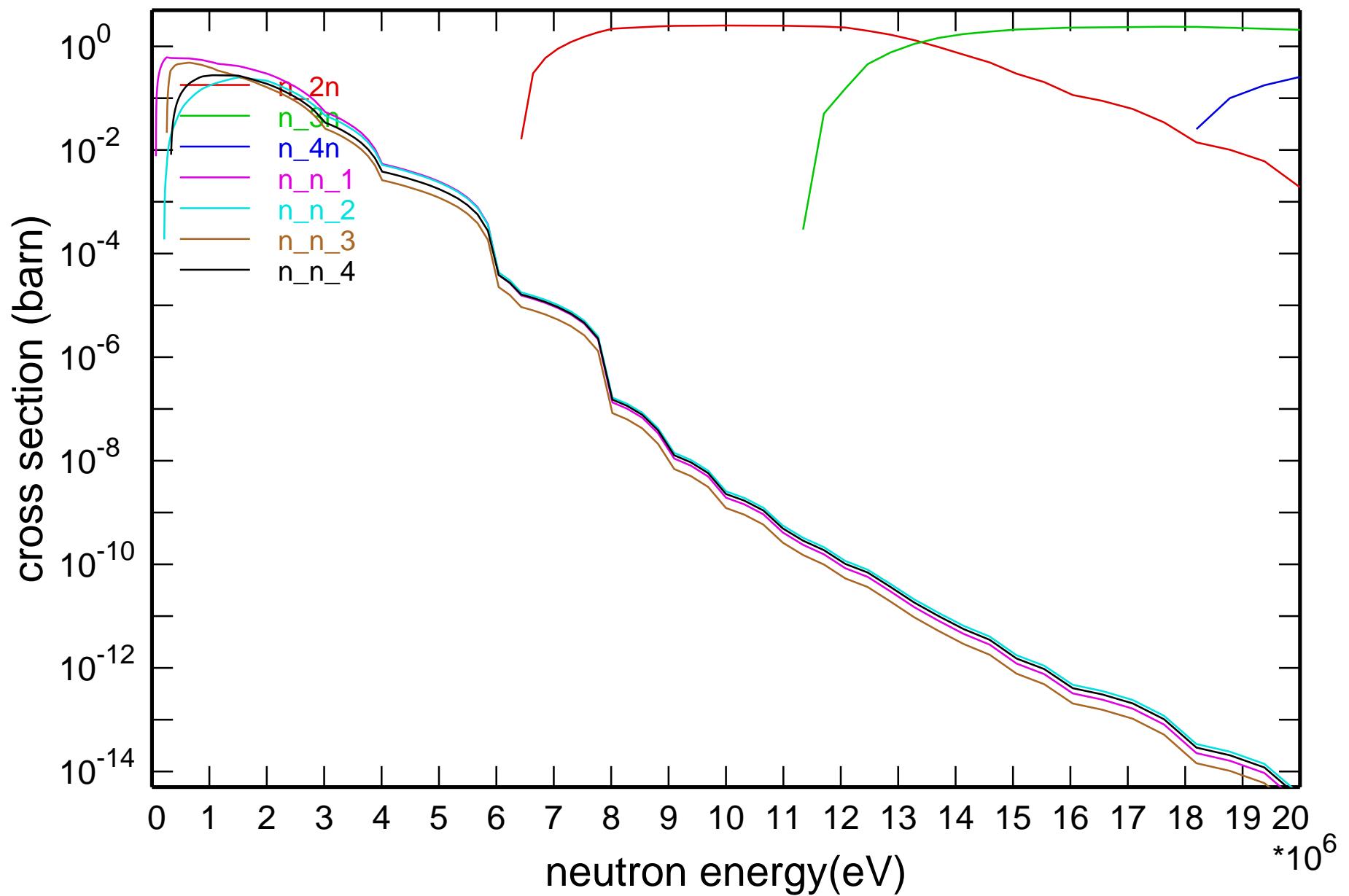


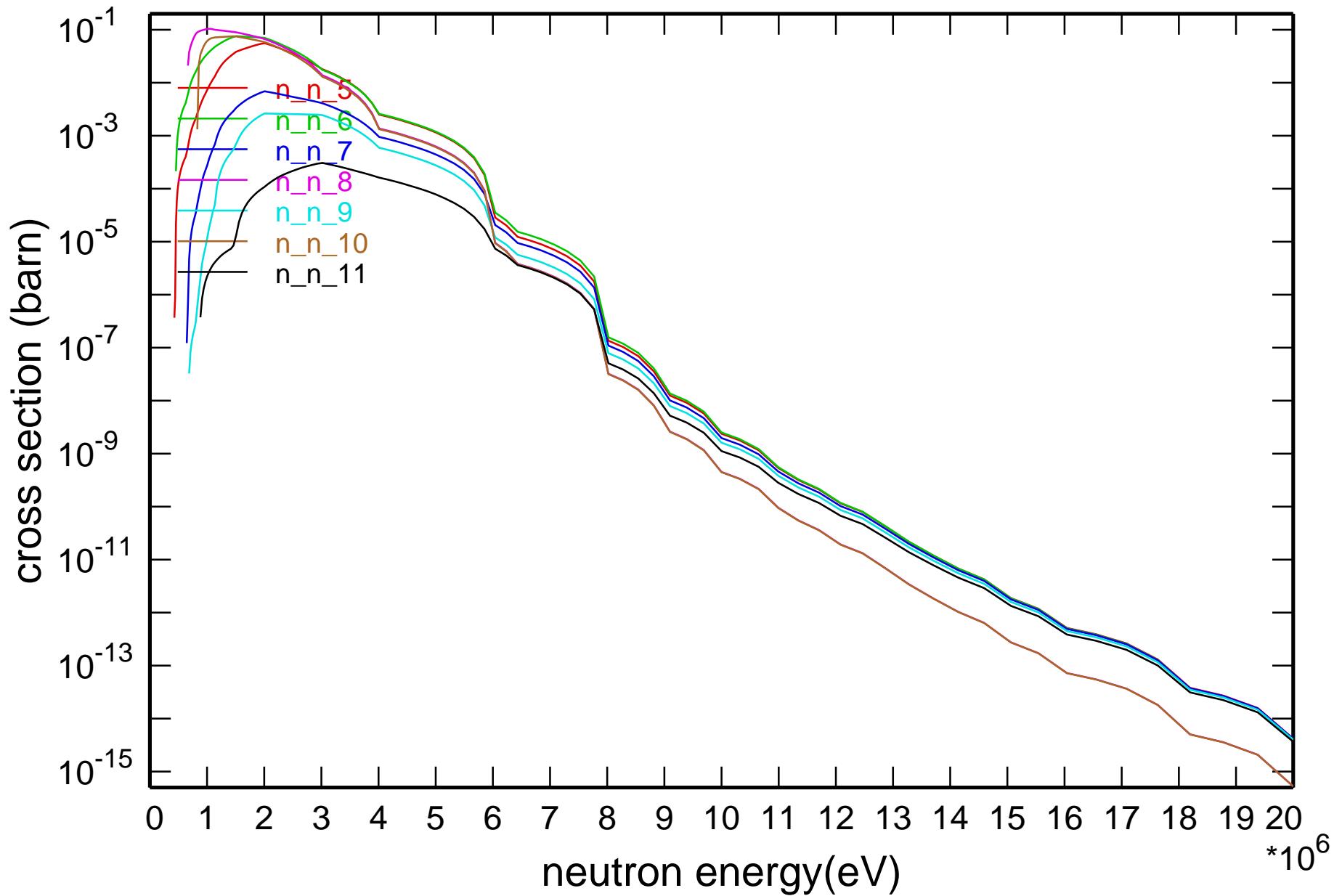
## Main Cross Sections



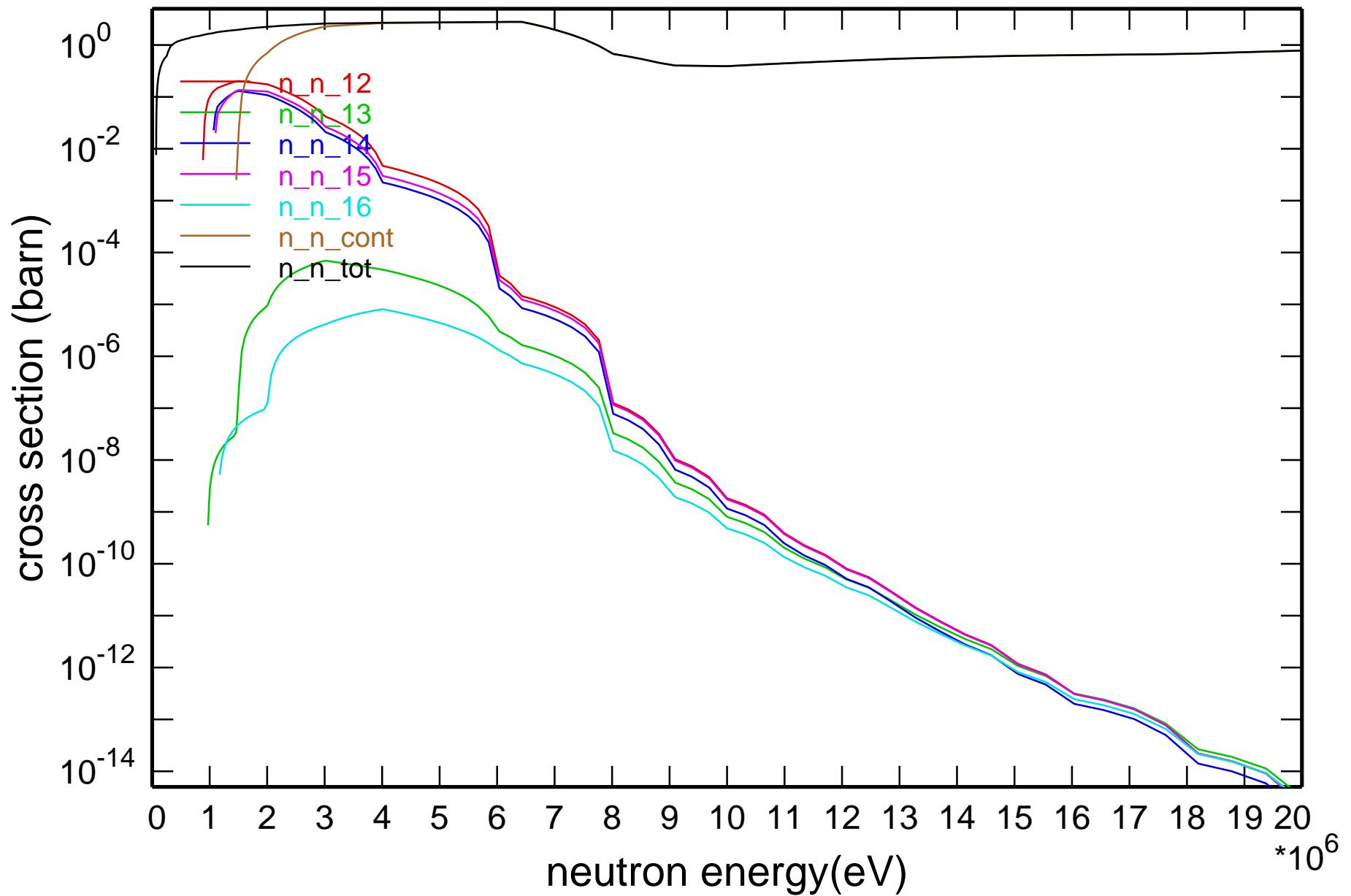
# Cross Section



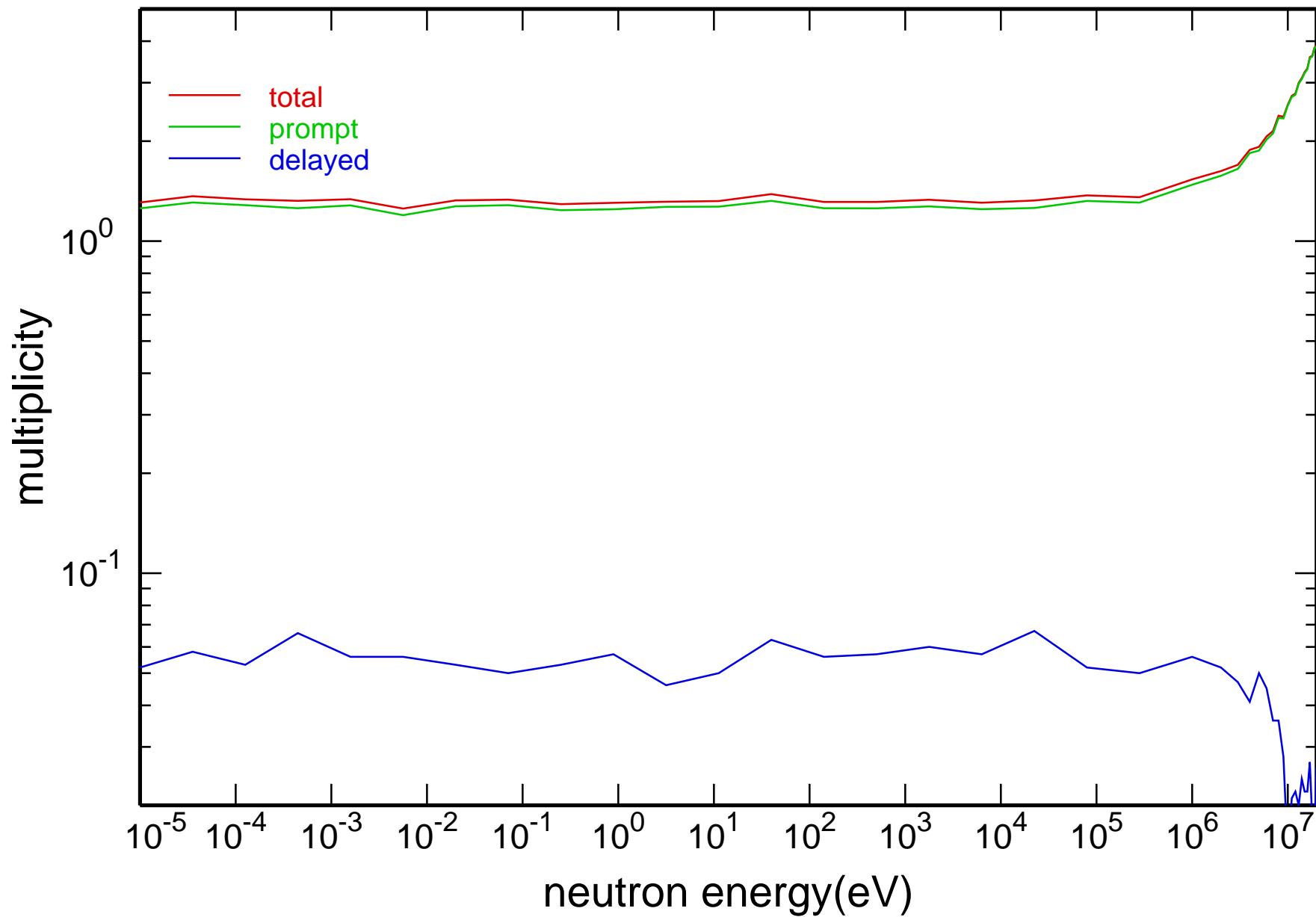
## Cross Section

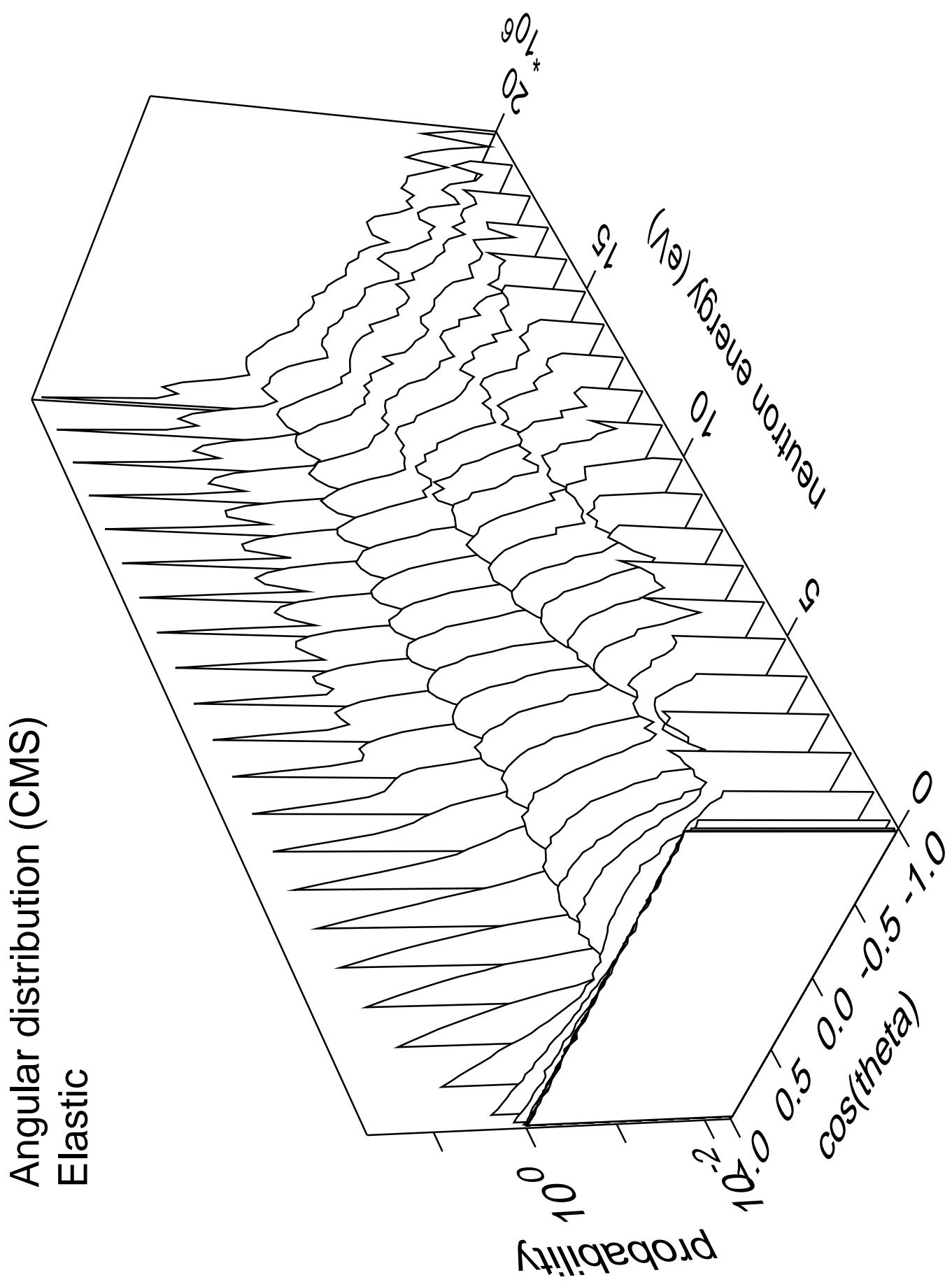


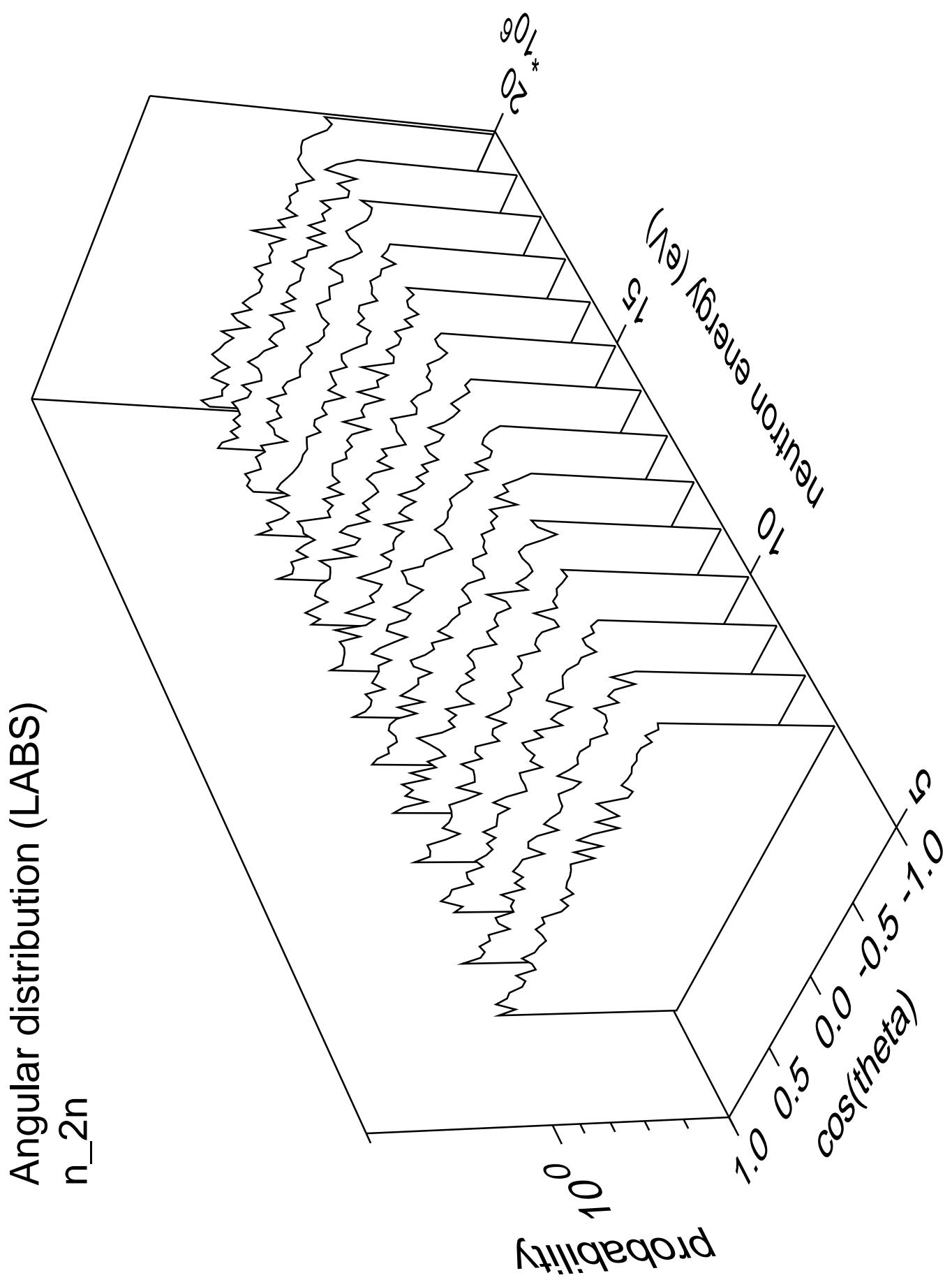
# Cross Section



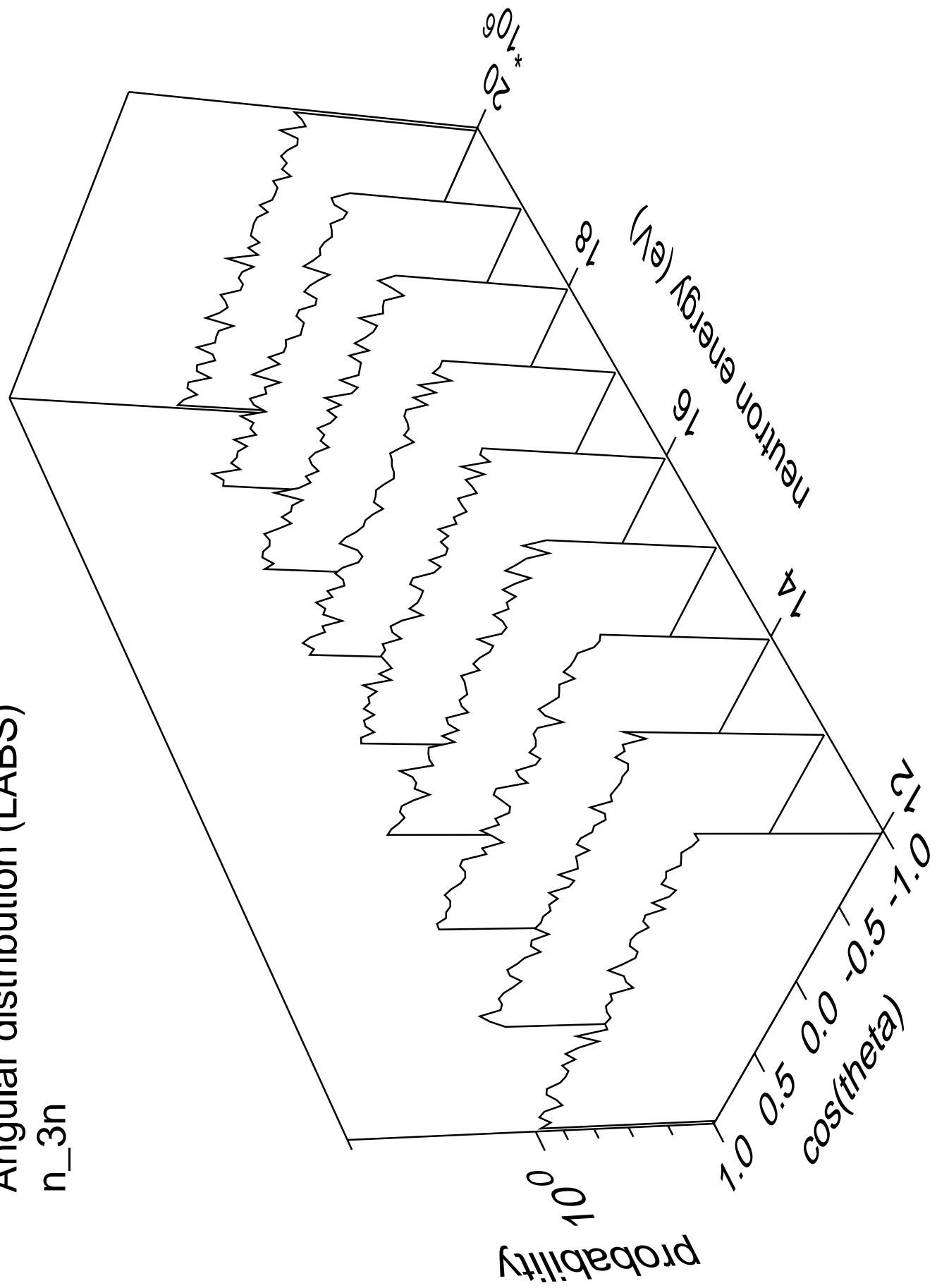
# neutron multiplicity for fission







Angular distribution (LABS)  
 $n_{3n}$



Angular distribution (LABS)  
 $n_{4n}$

Probability

$10^0$

1.0

0.5

0.0

-0.5

-1.0

$\cos(\theta)$

0.0

-0.5

-1.0

0.5

1.0

Neutron energy (eV)

$20.0 \times 10^6$

19.5

19.0

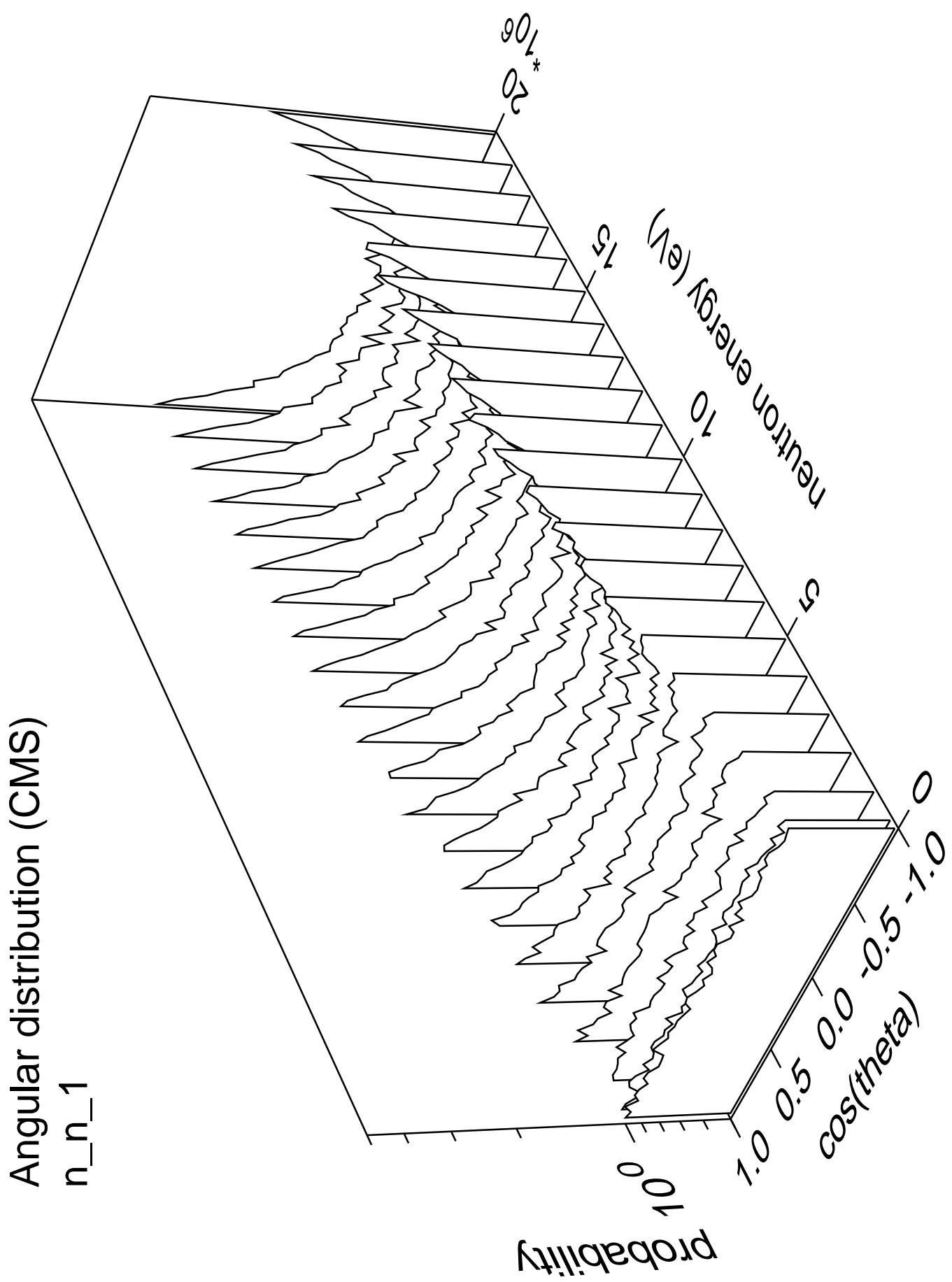
18.5

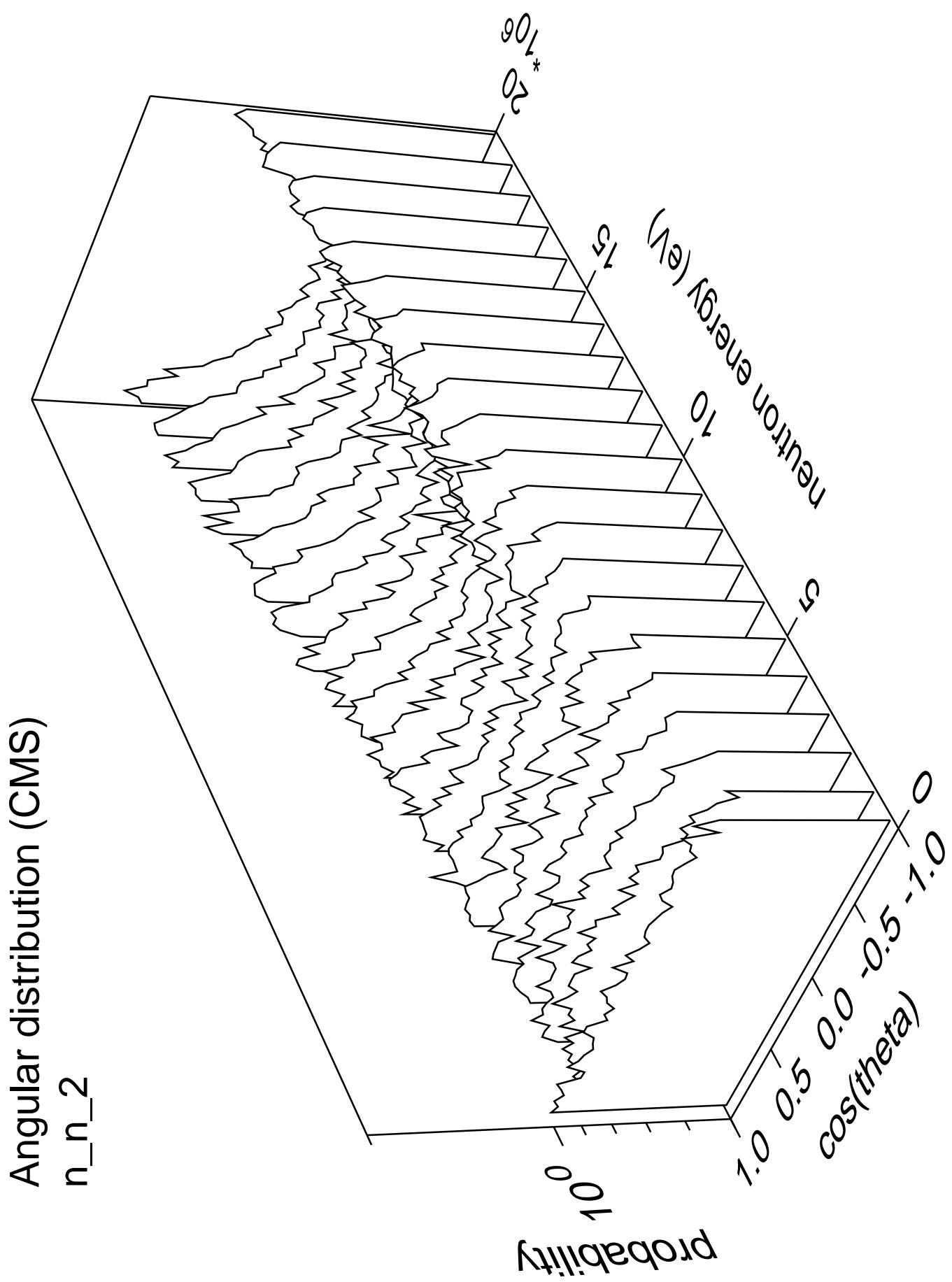
18.0

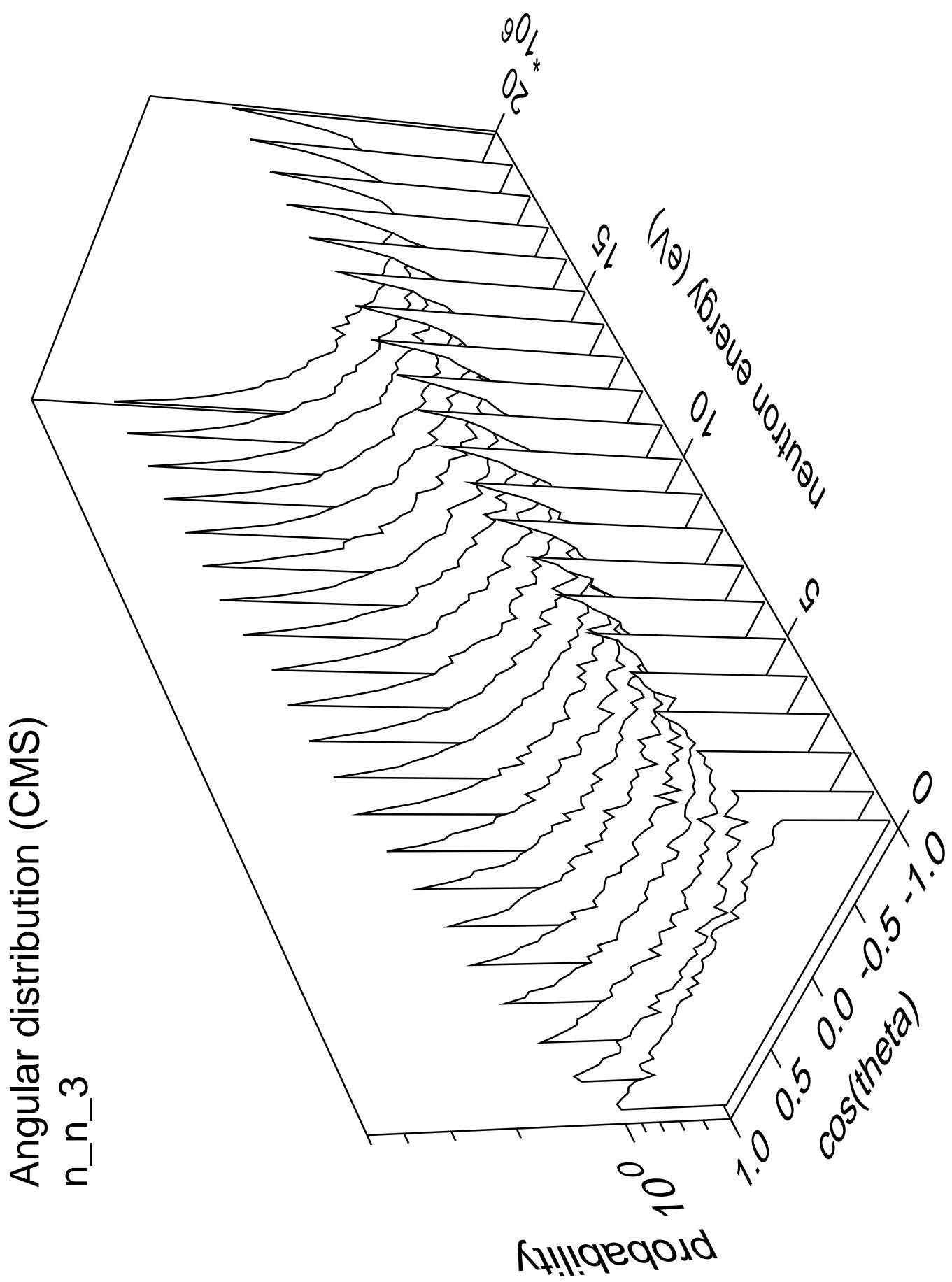
17.5

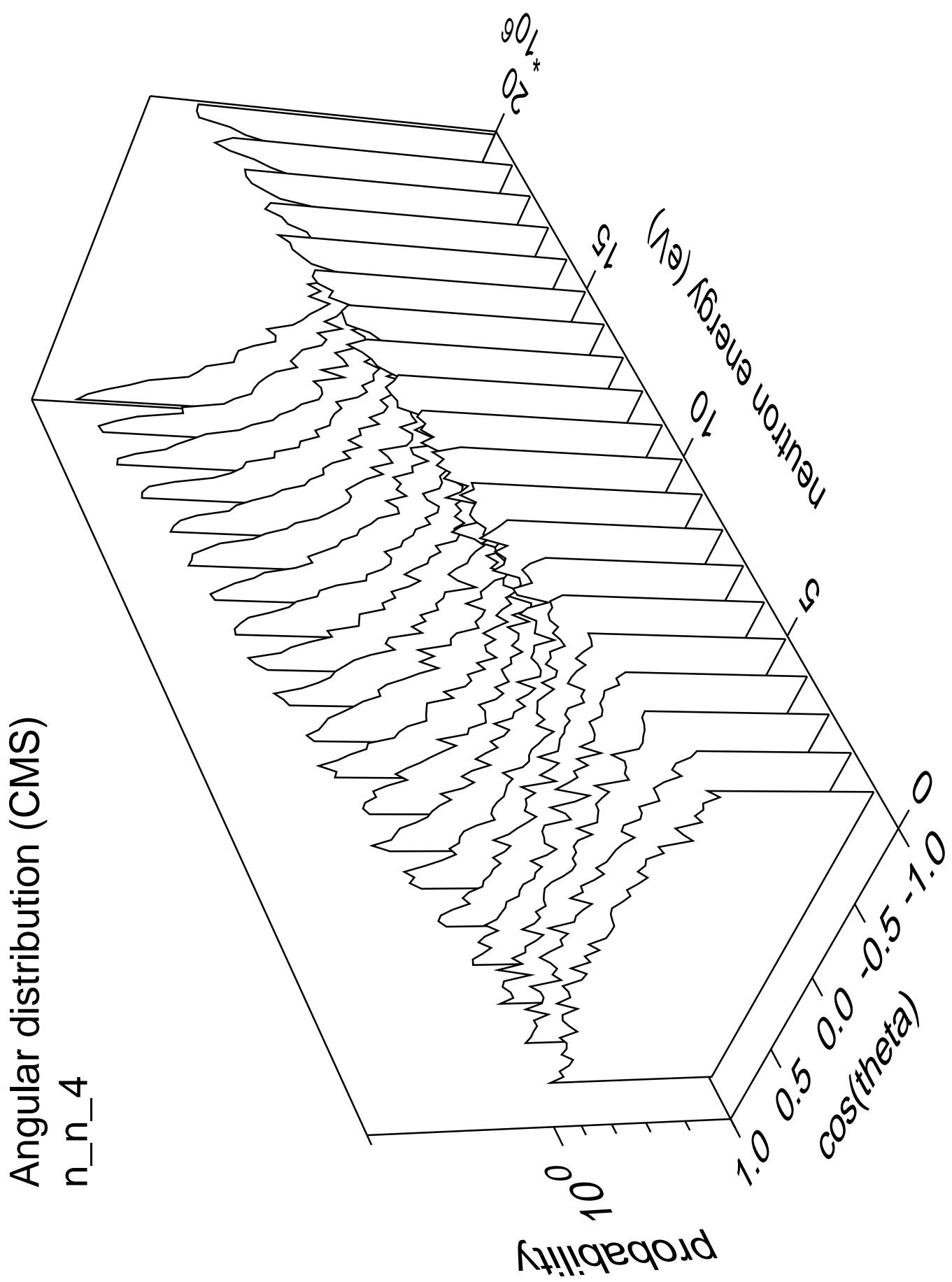
17.0

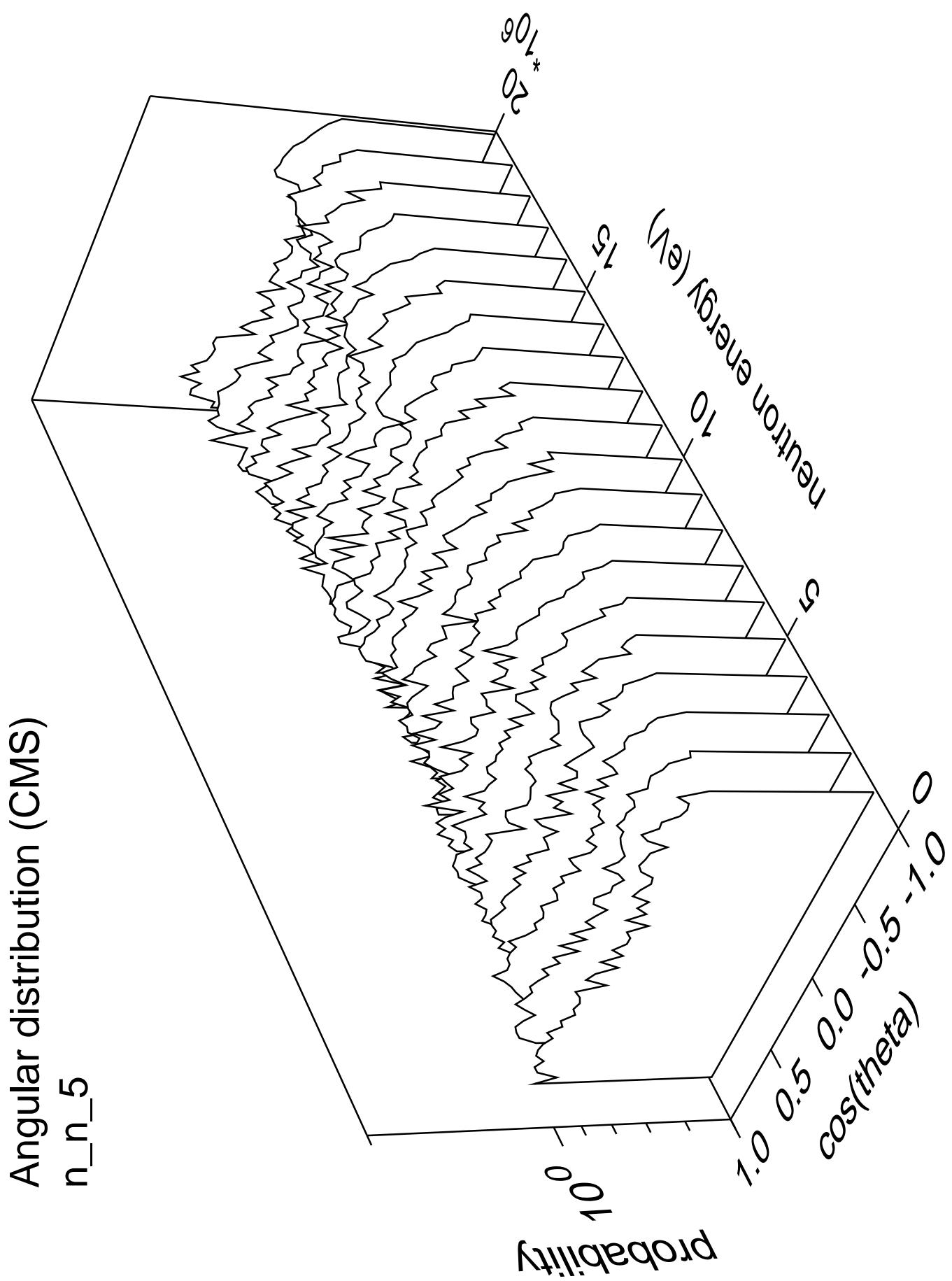
16.5

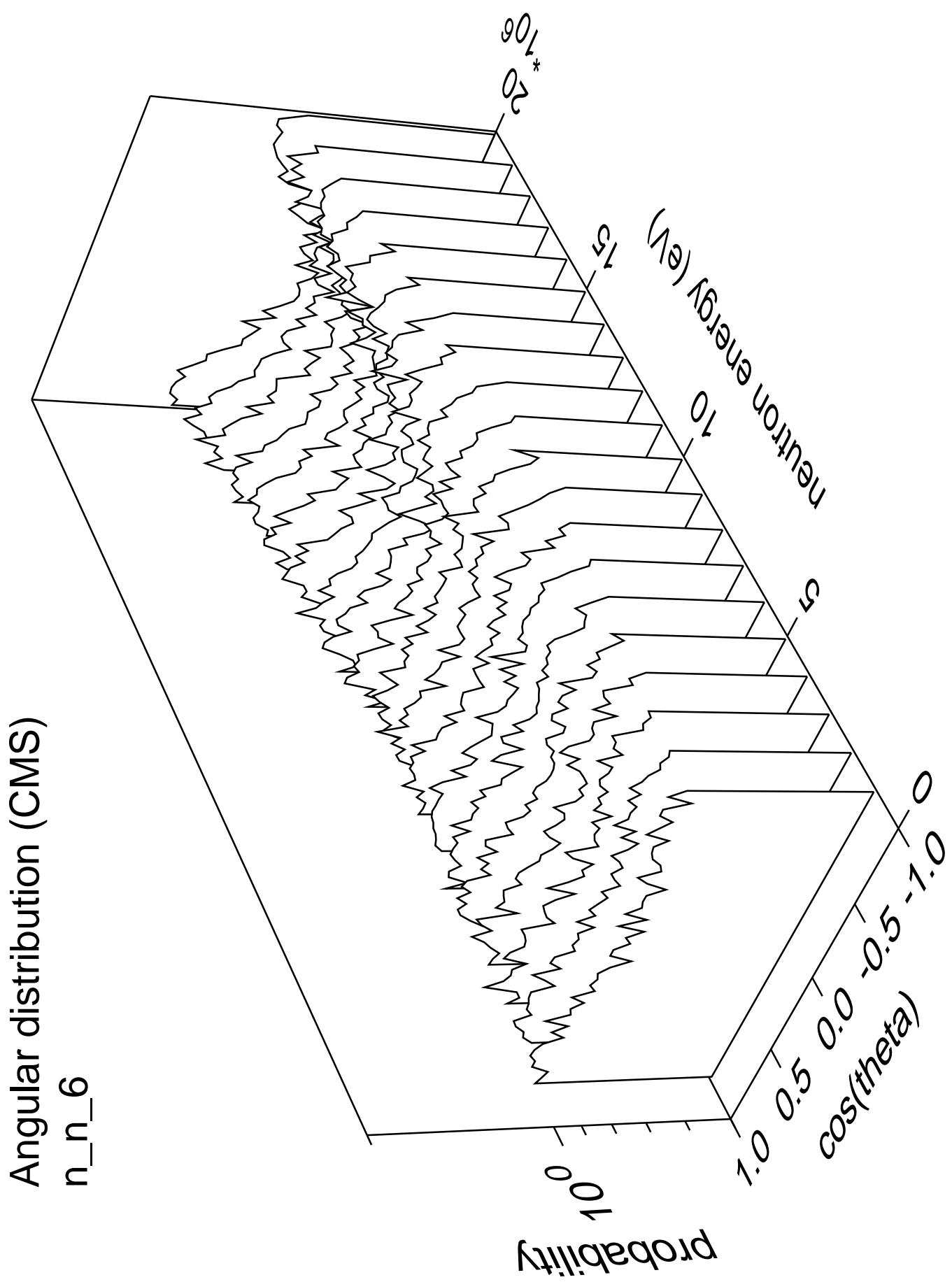


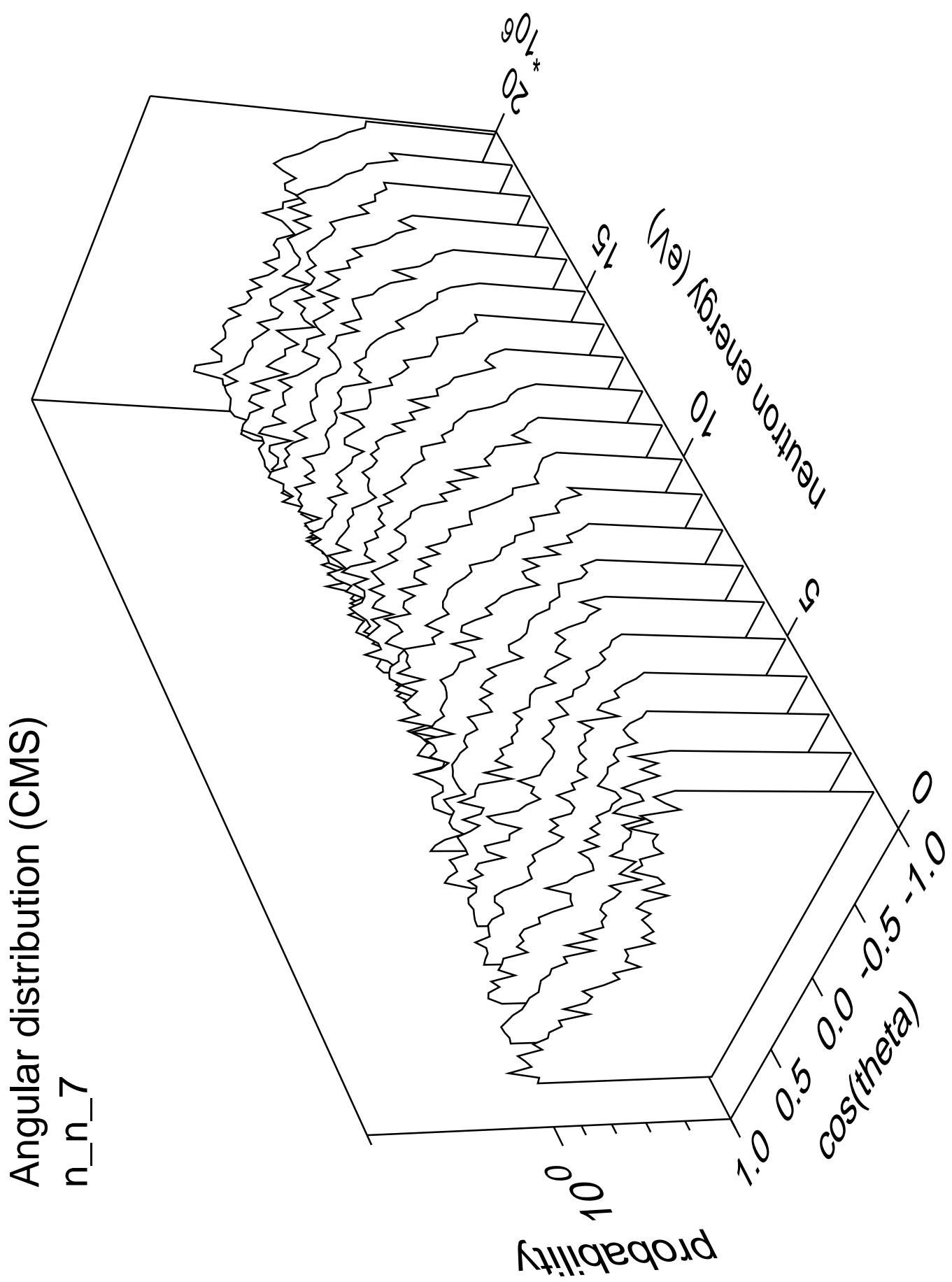


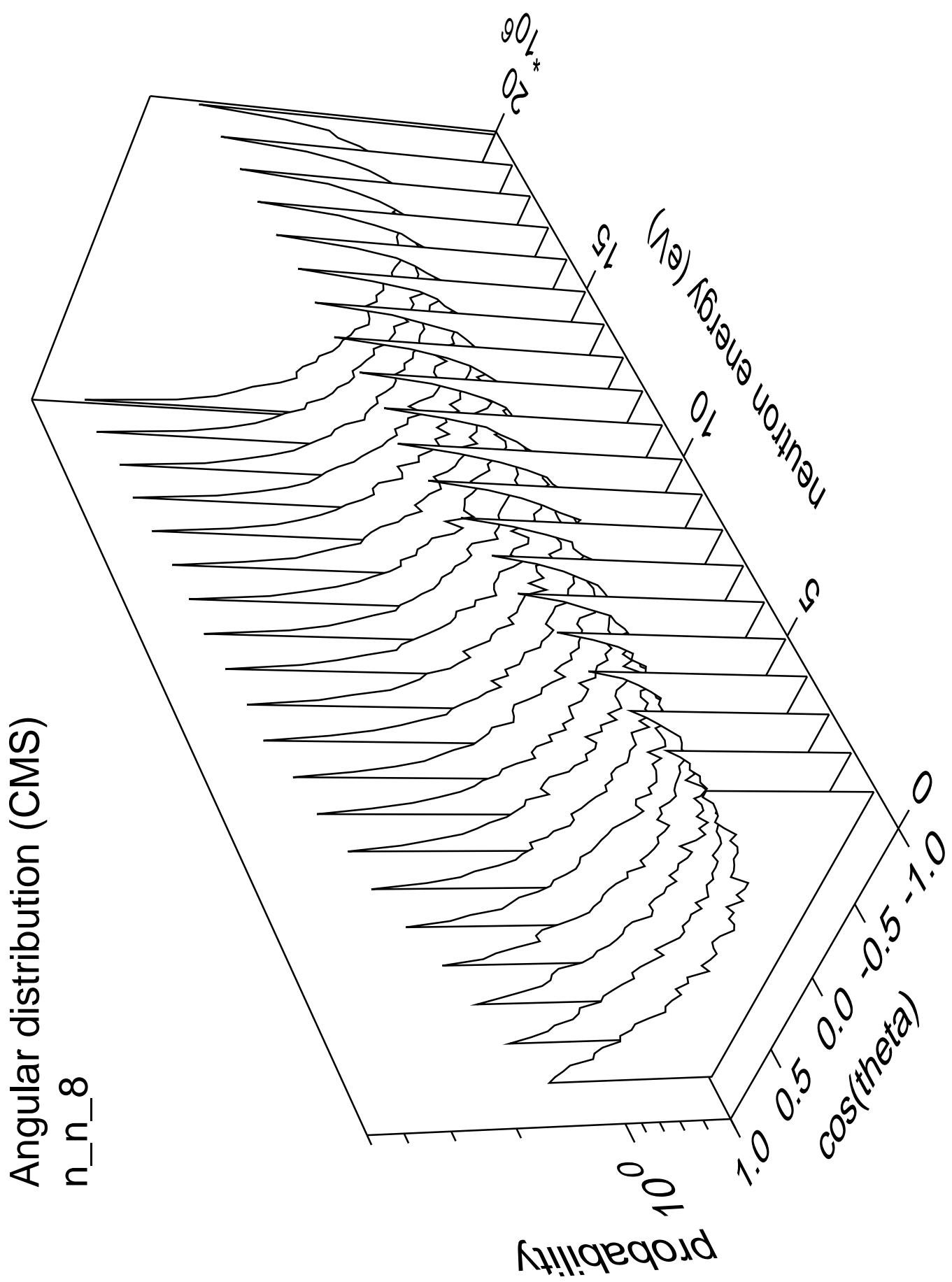


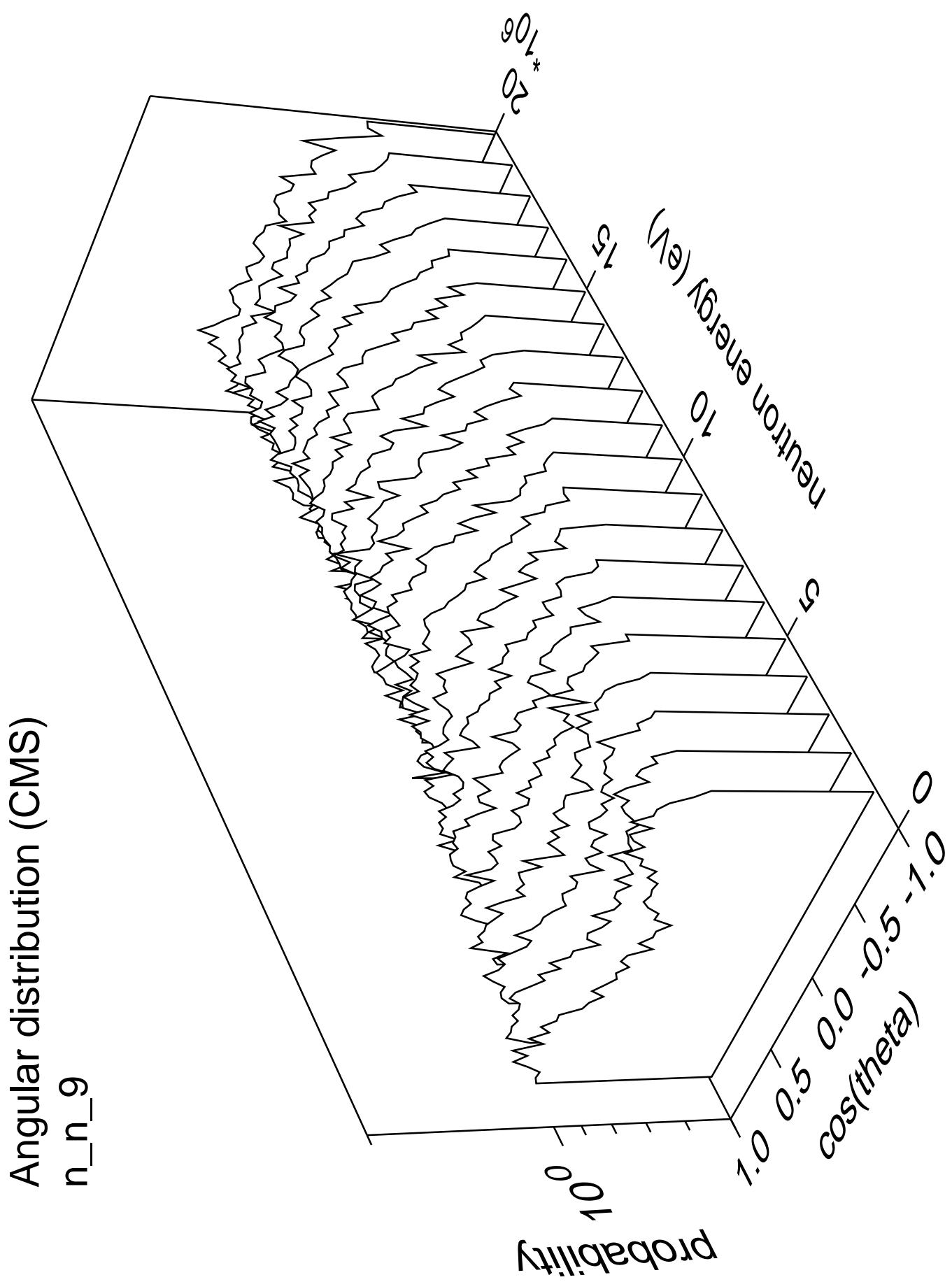


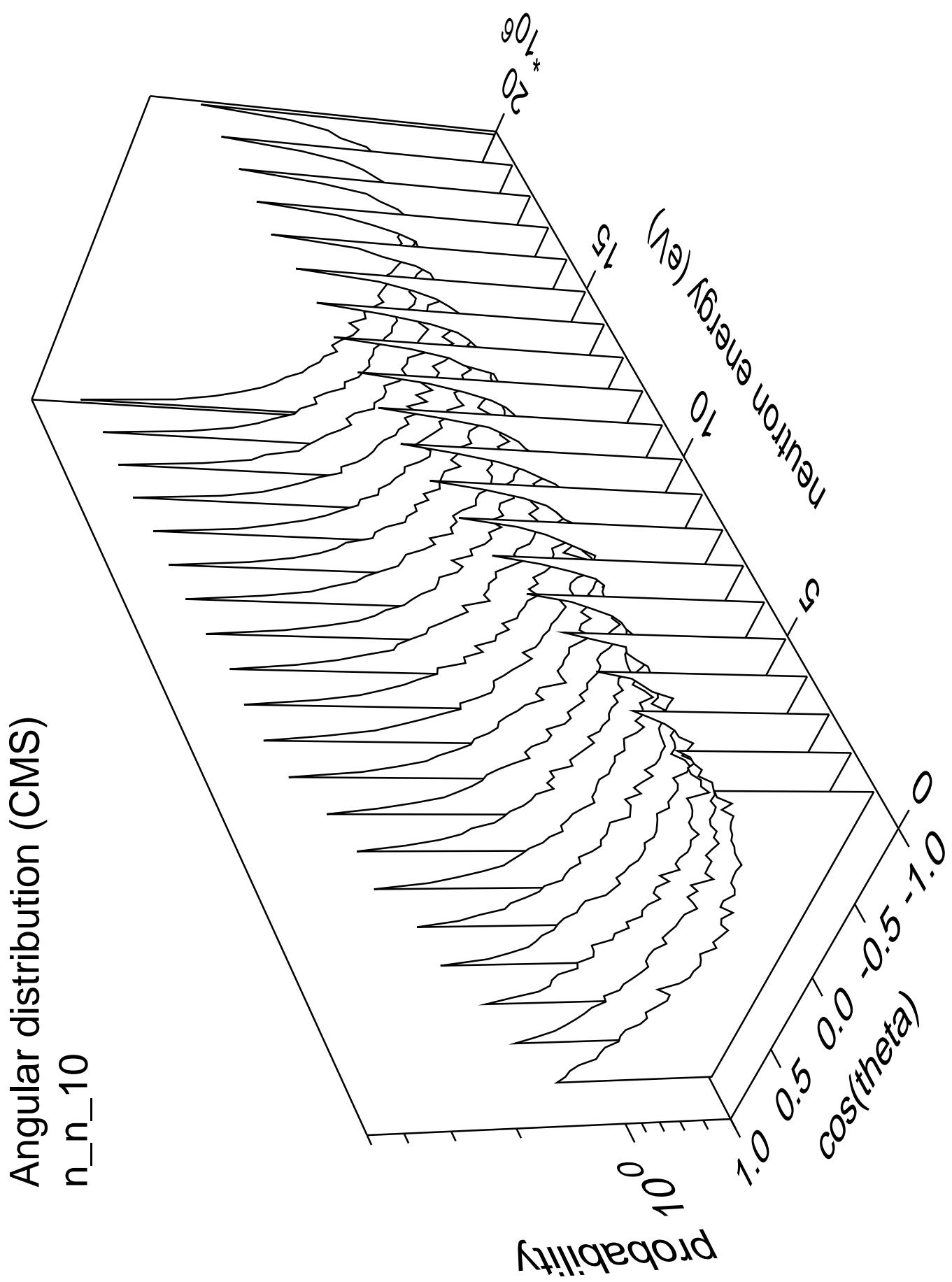


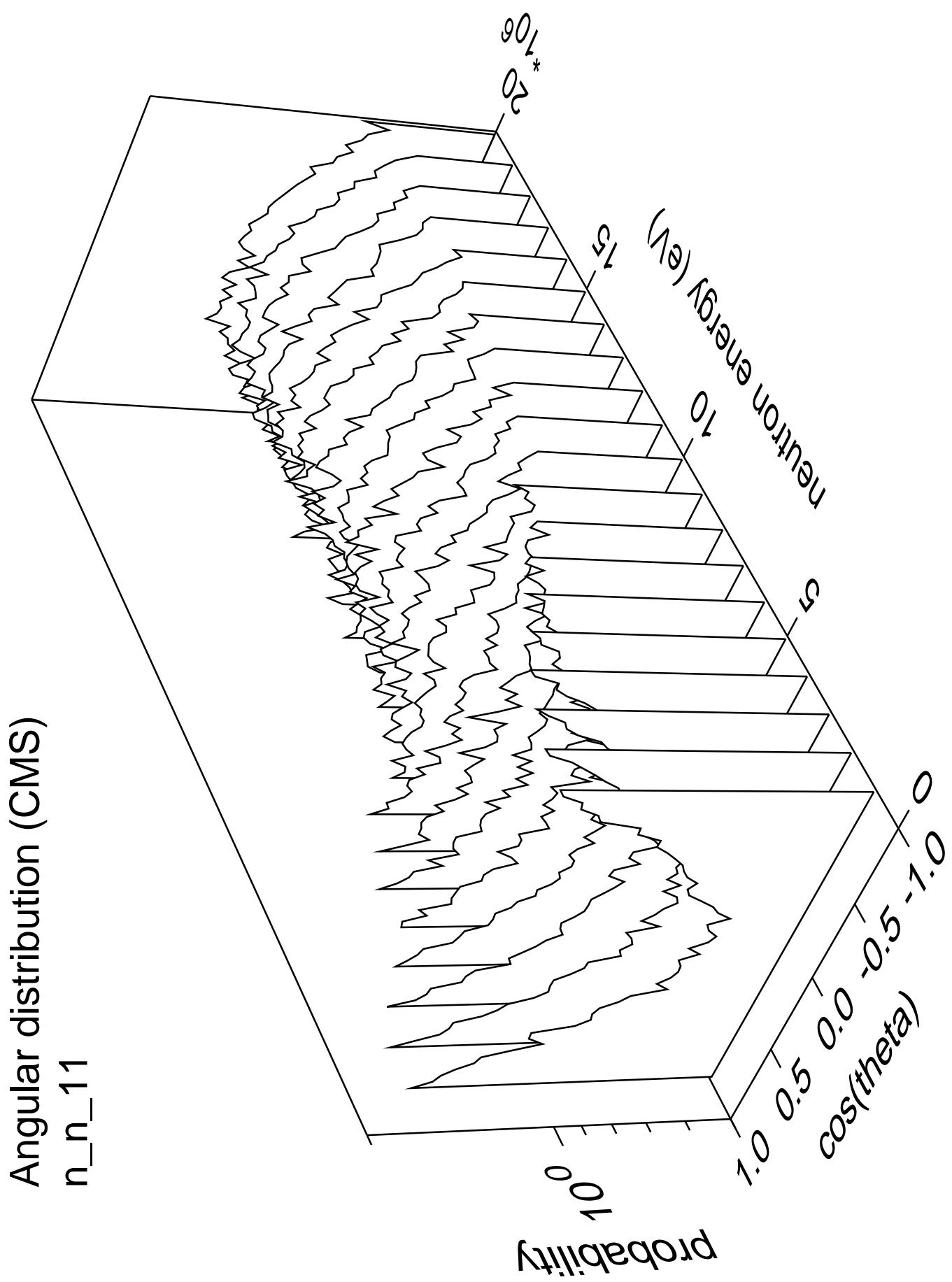


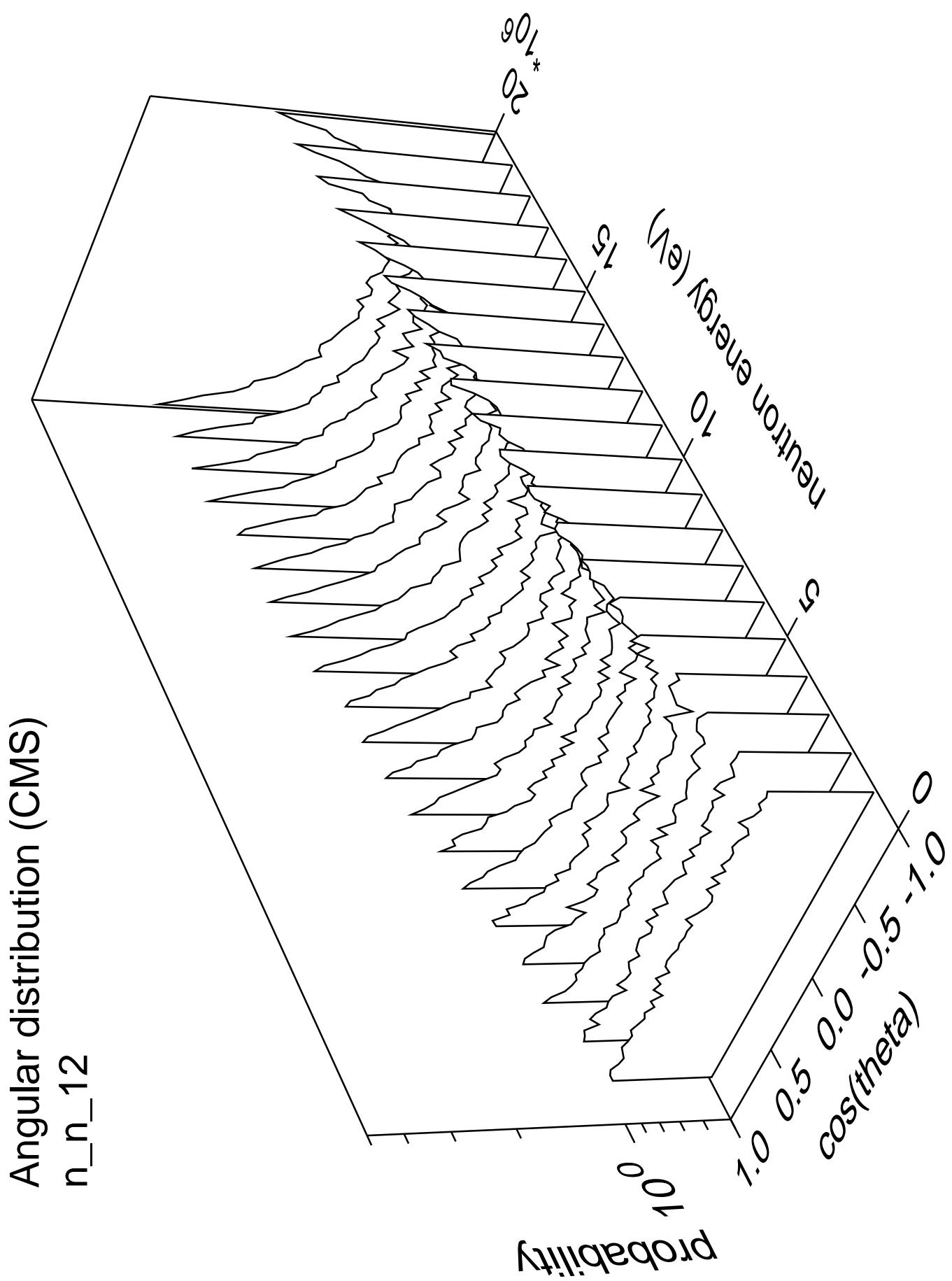




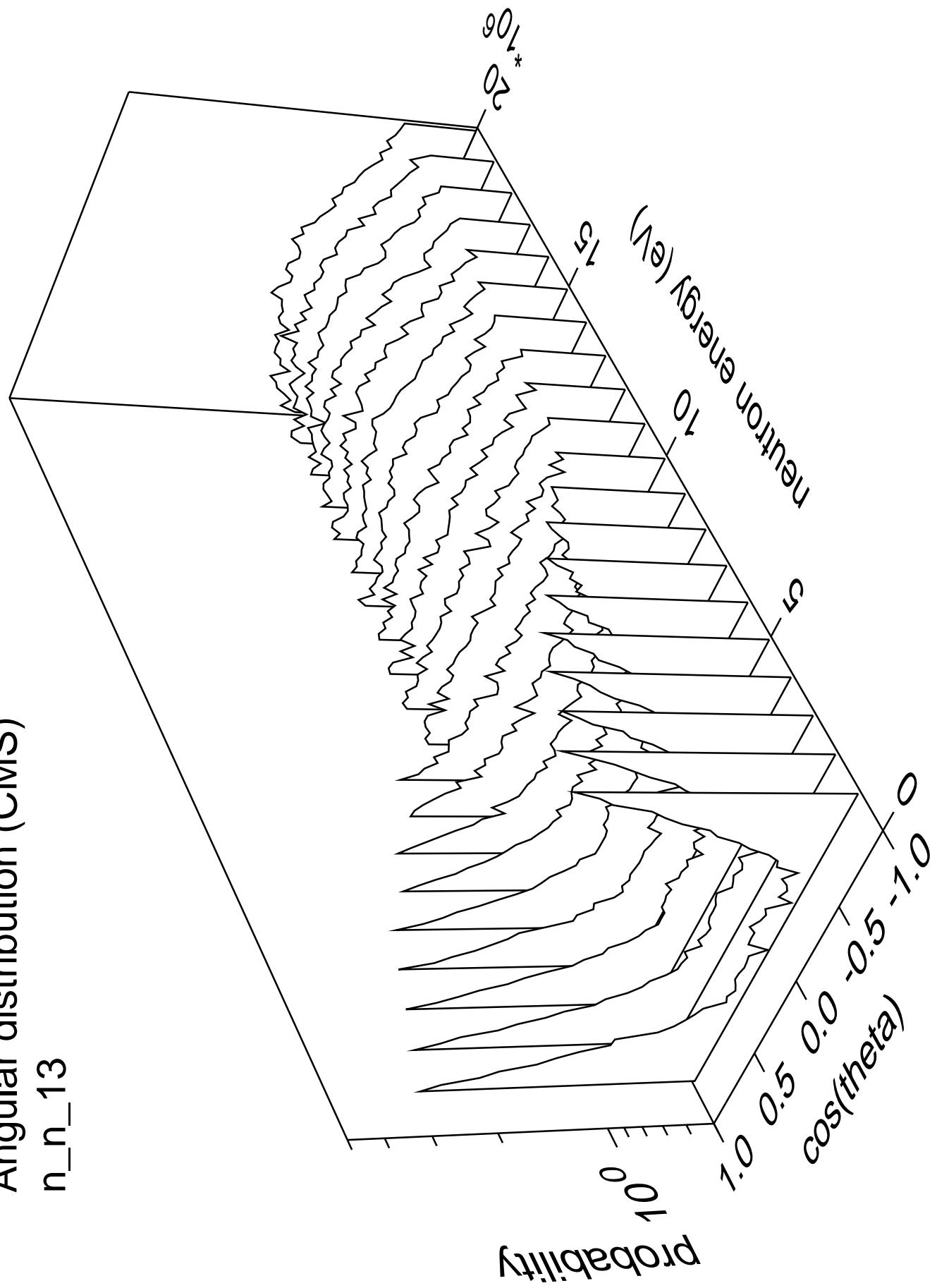


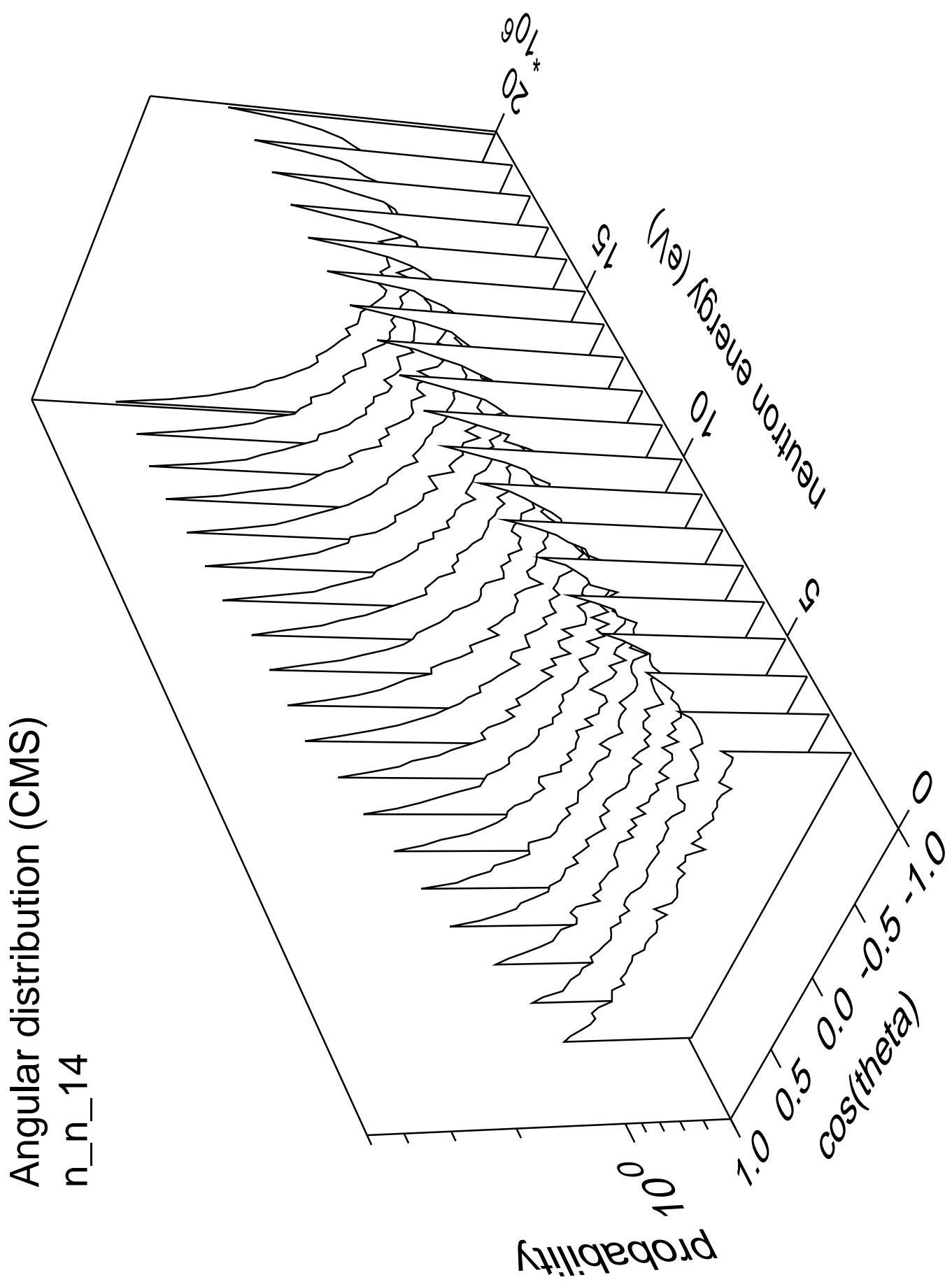


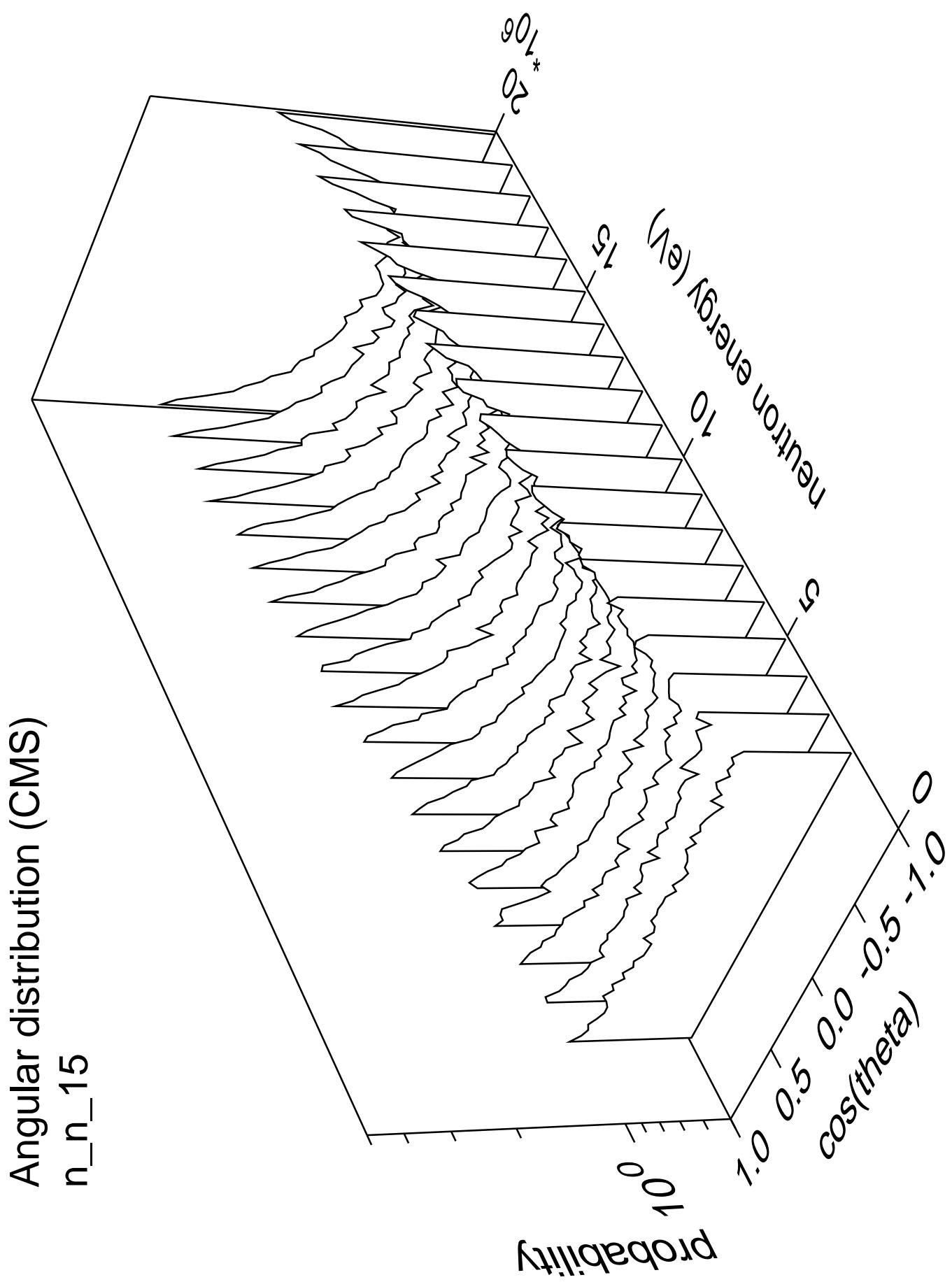




Angular distribution (CMS)  
n\_n\_13

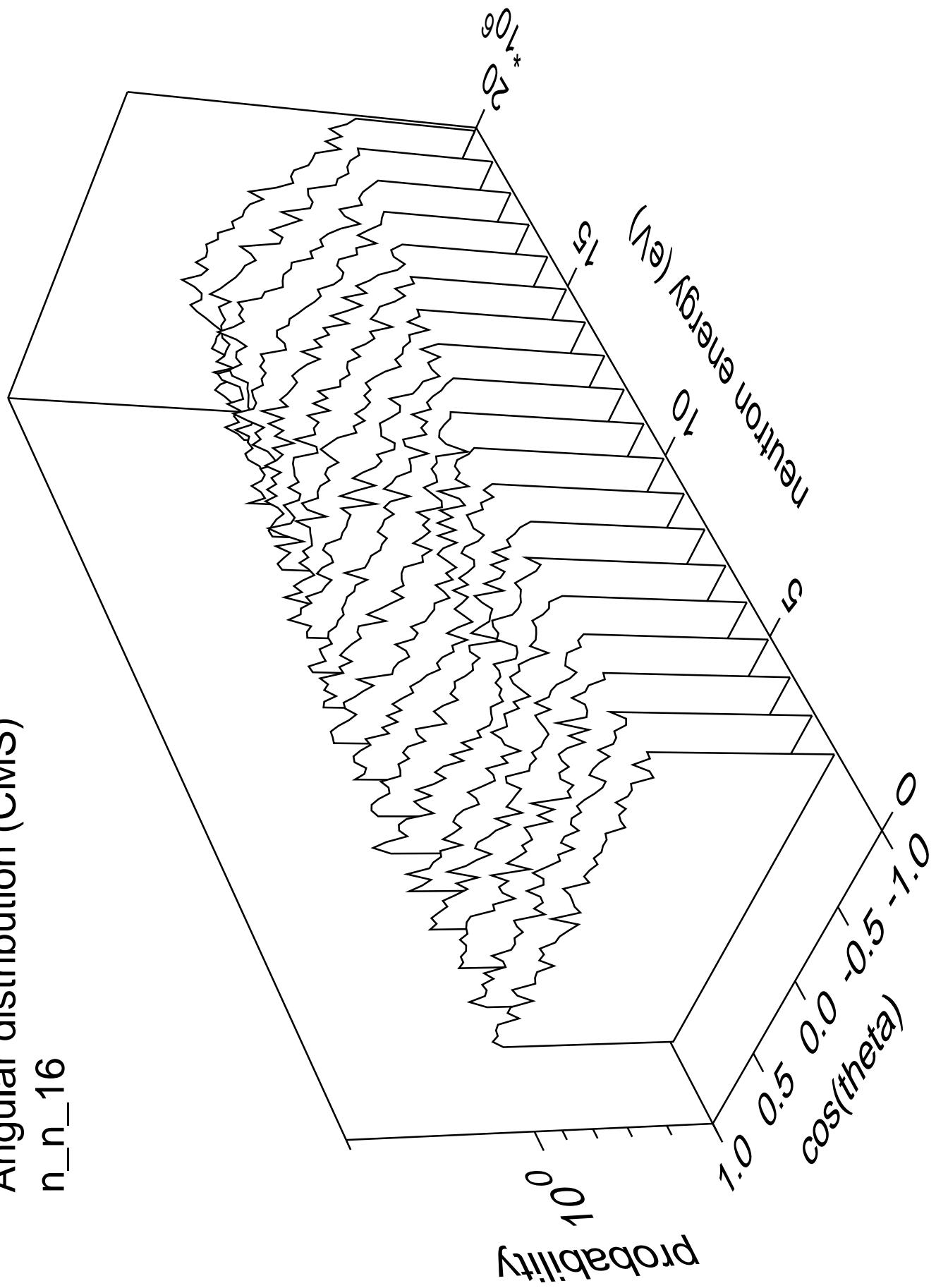


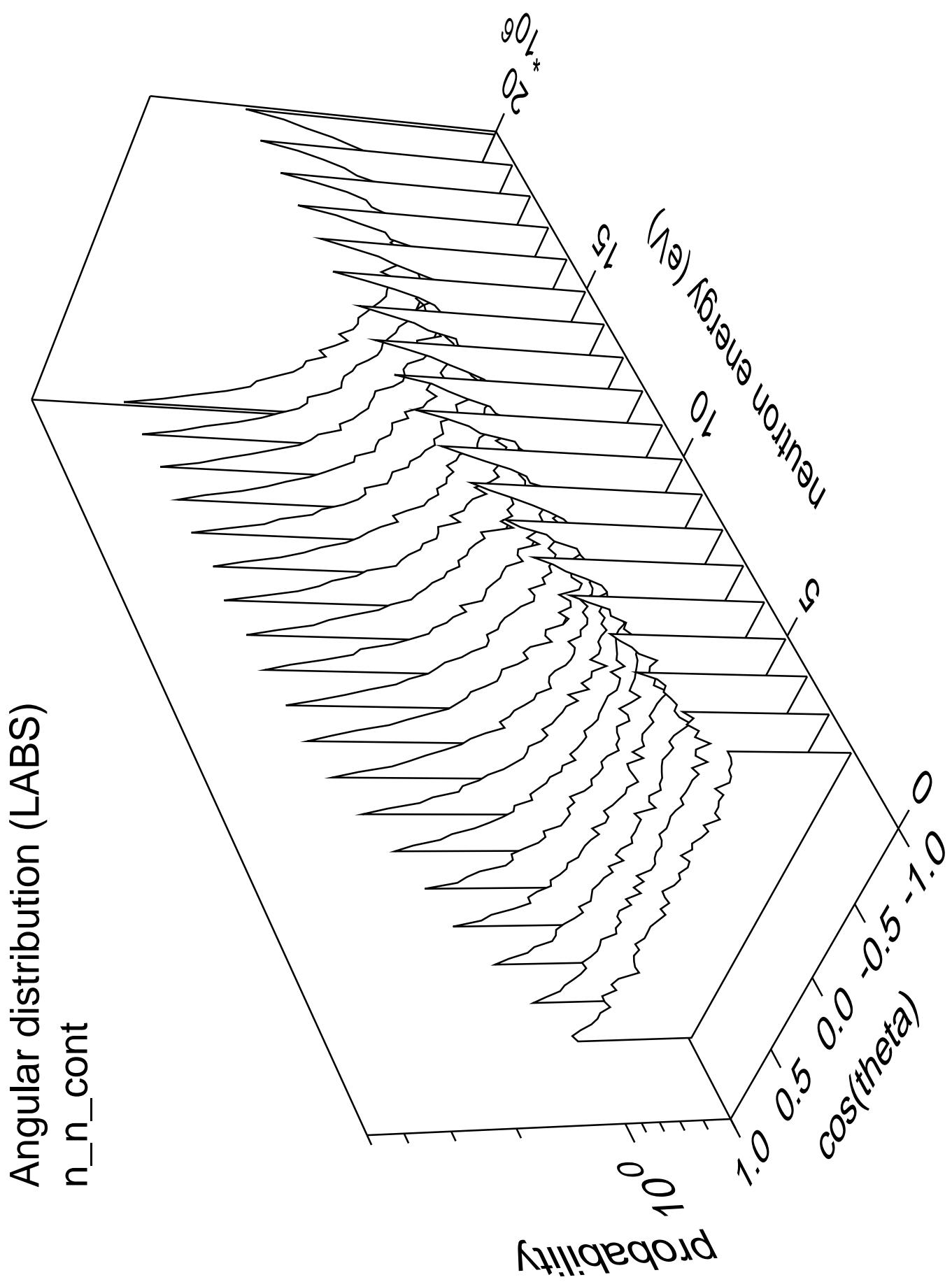




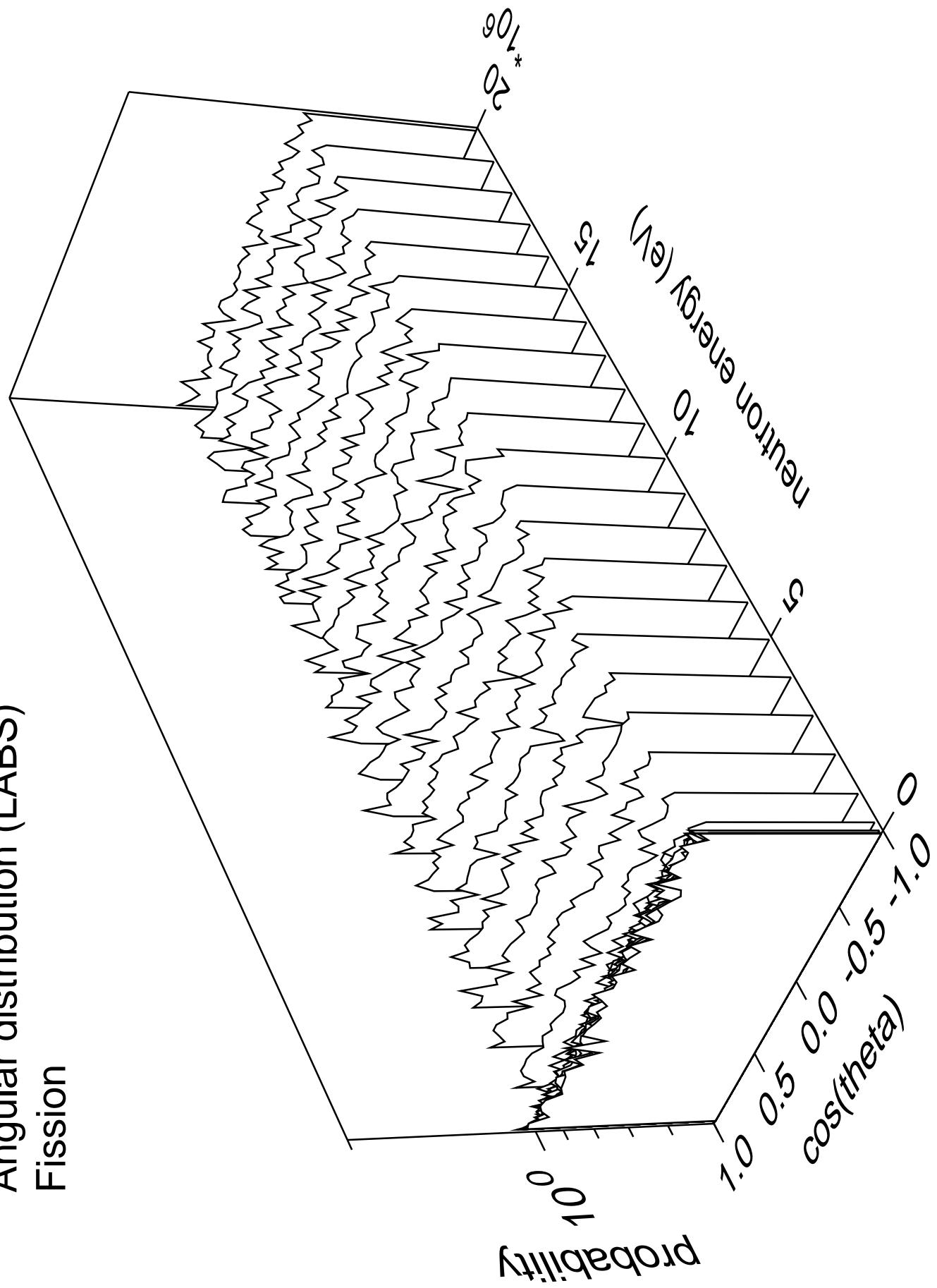
Angular distribution (CMS)

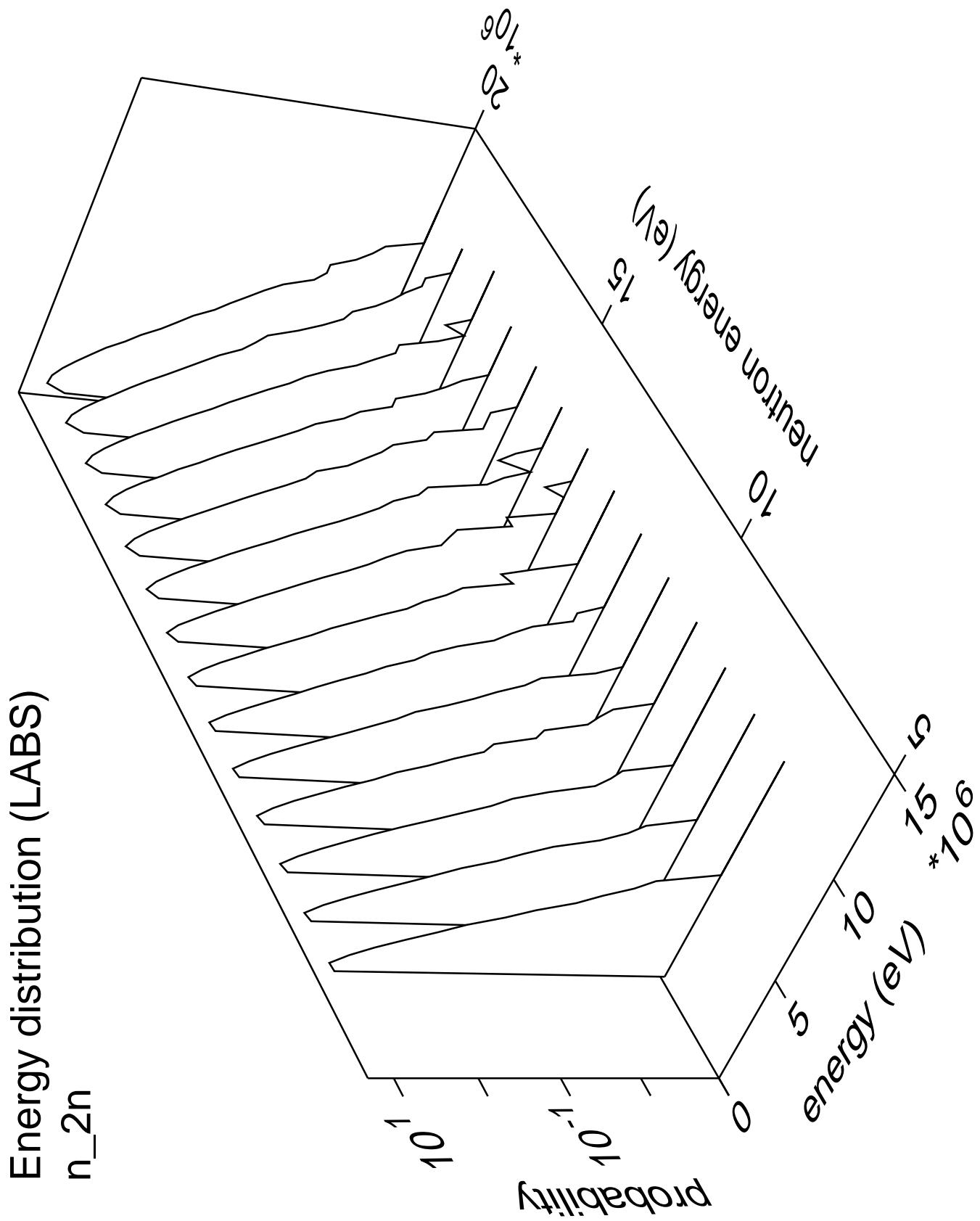
n\_n\_16

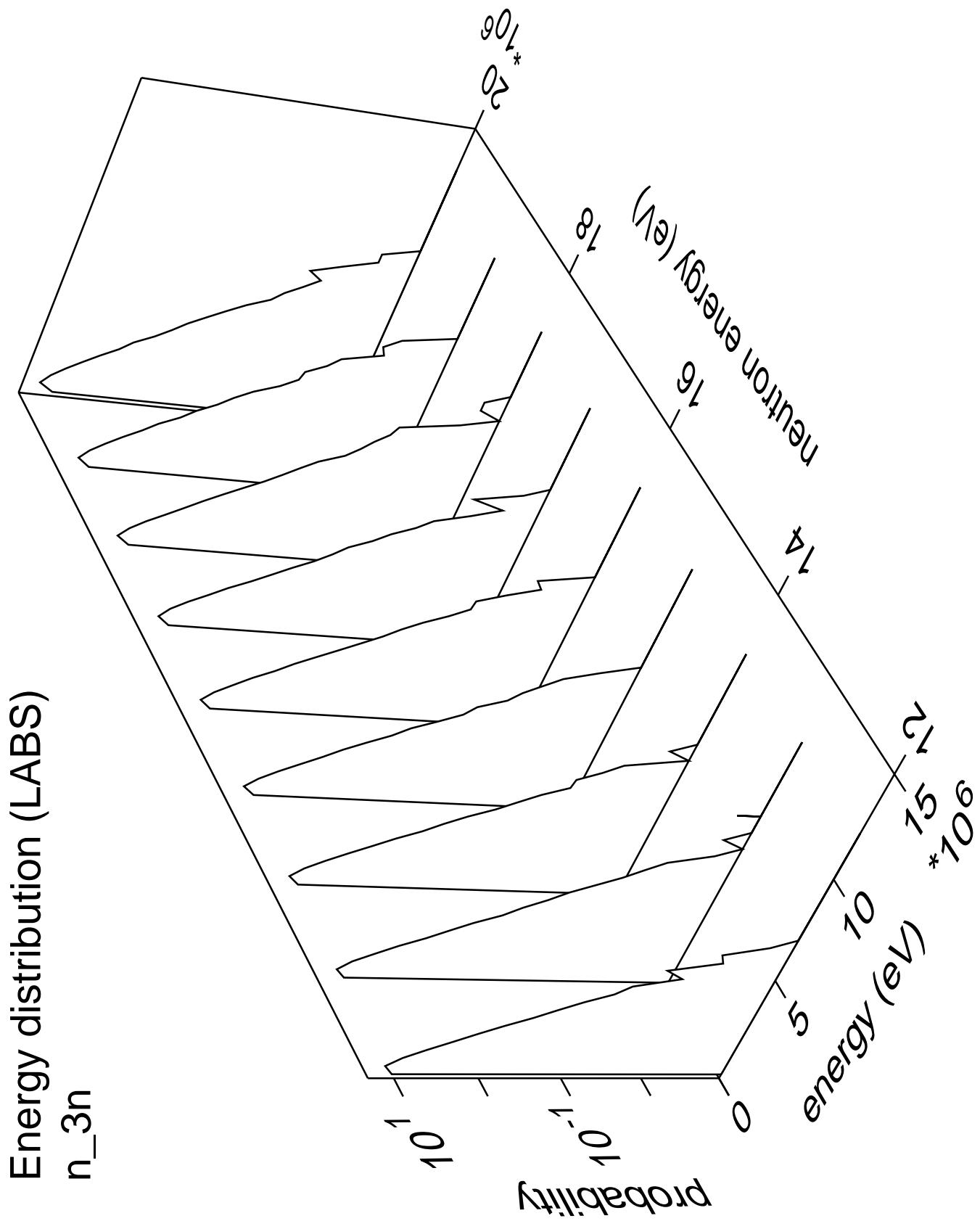


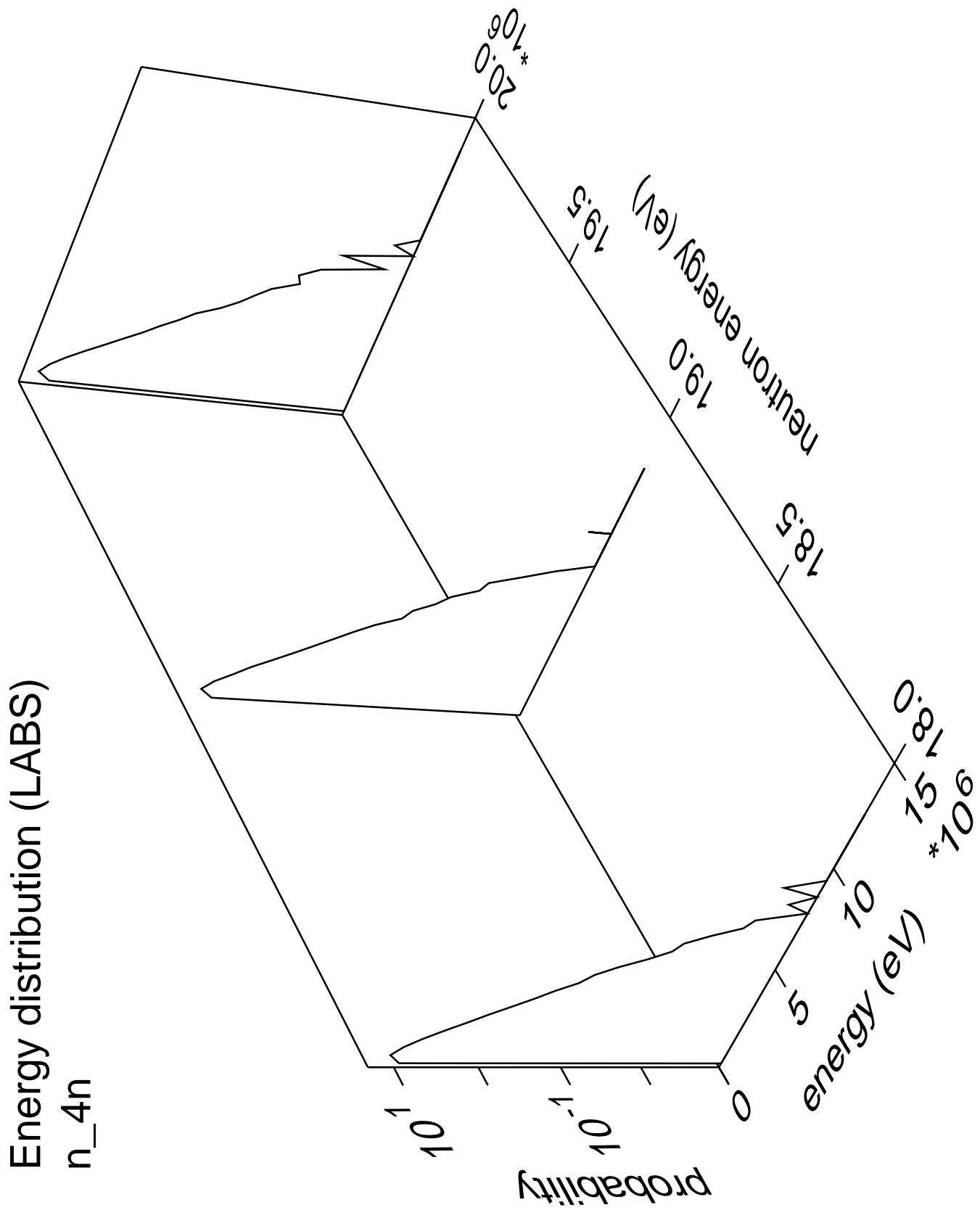


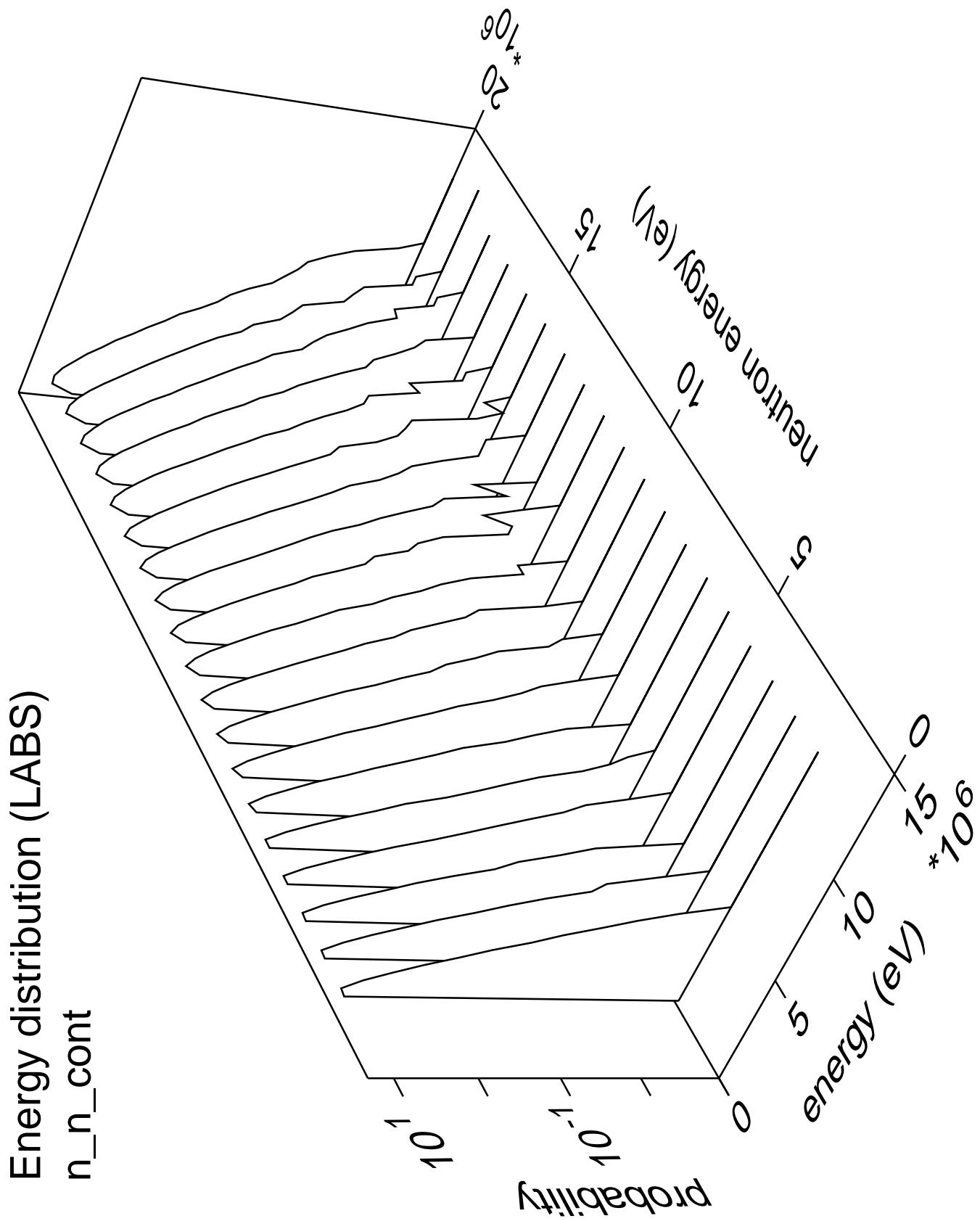
# Angular distribution (LABS) Fission











Energy distribution (CMS)  
Fission prompt

