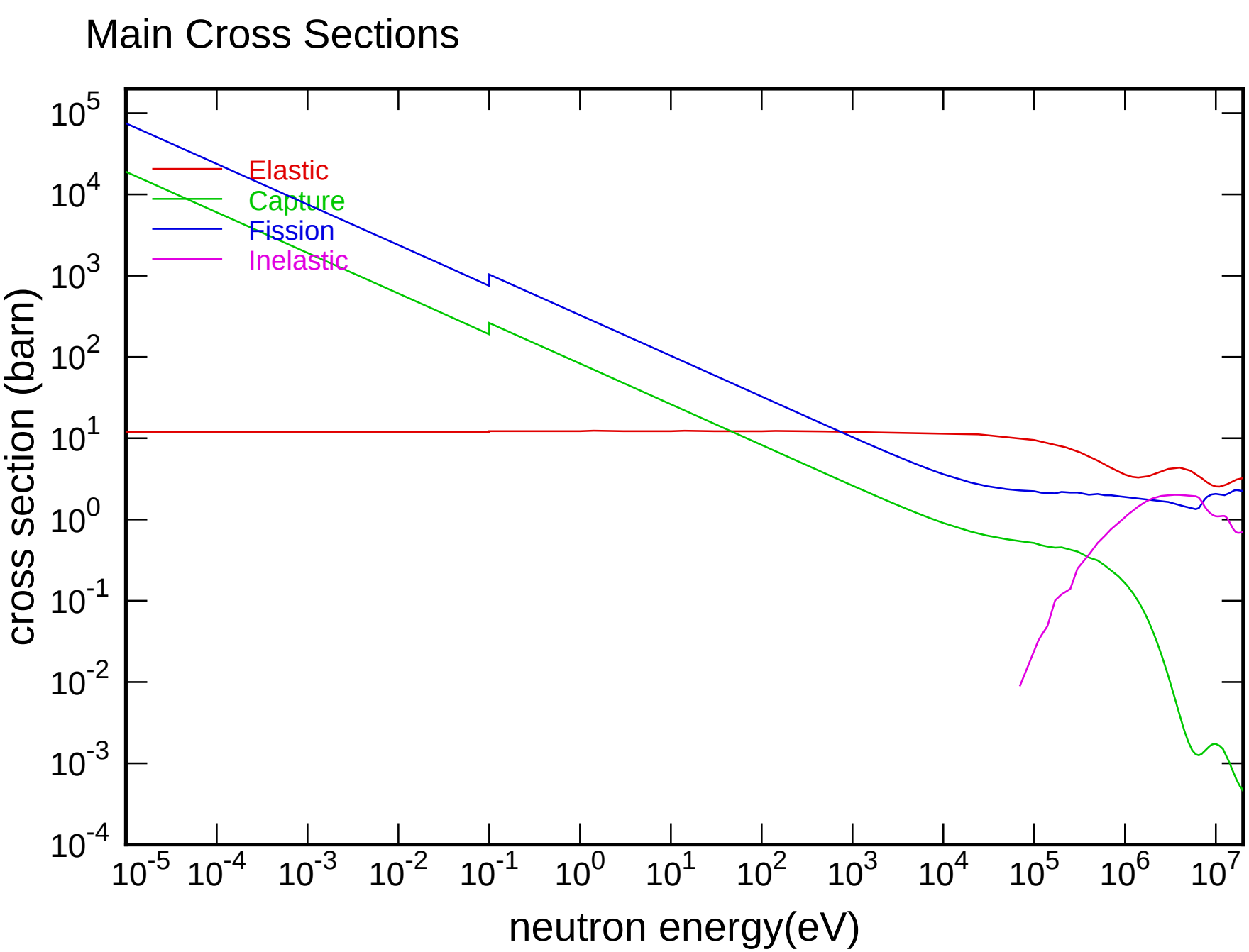
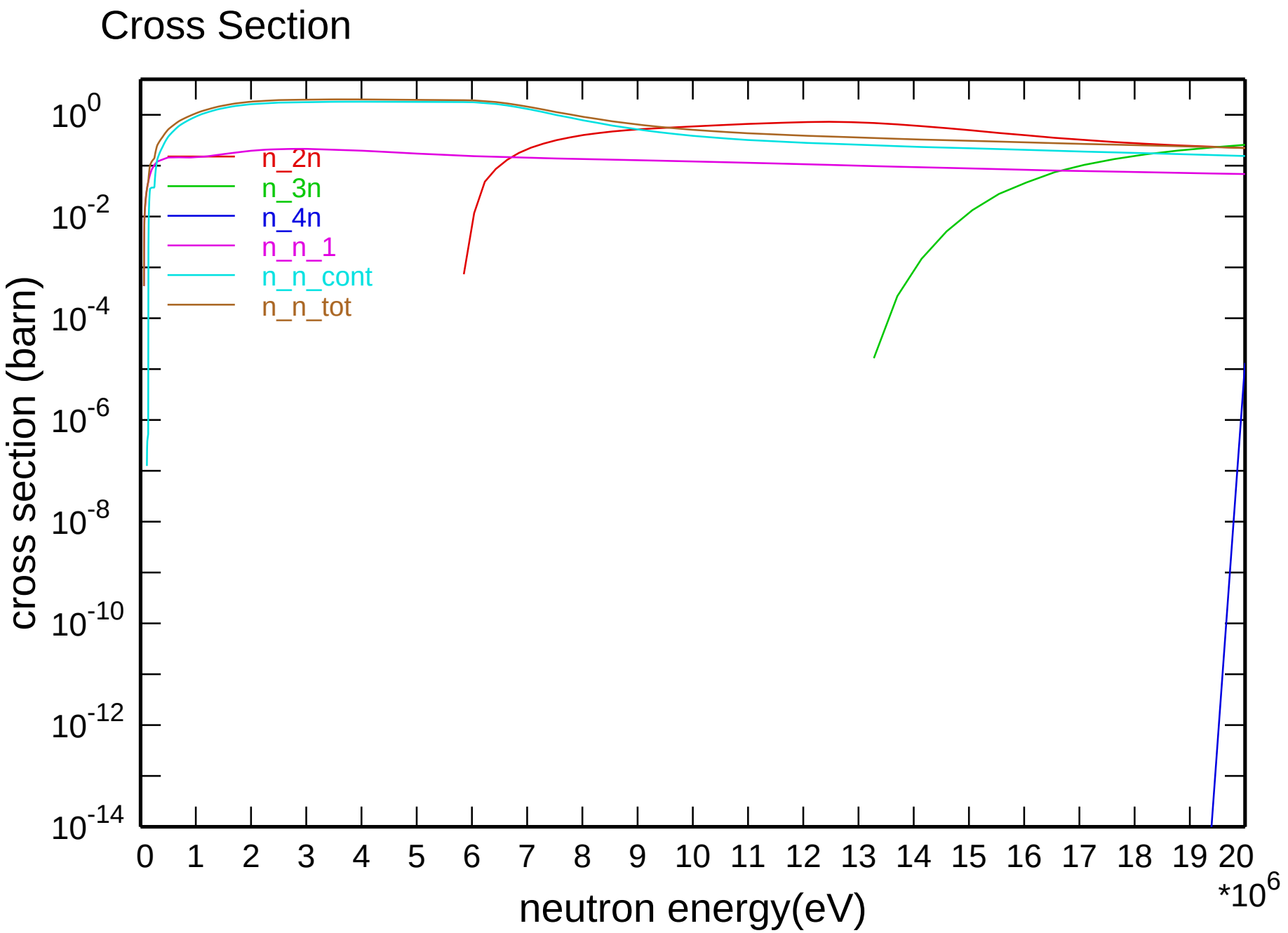
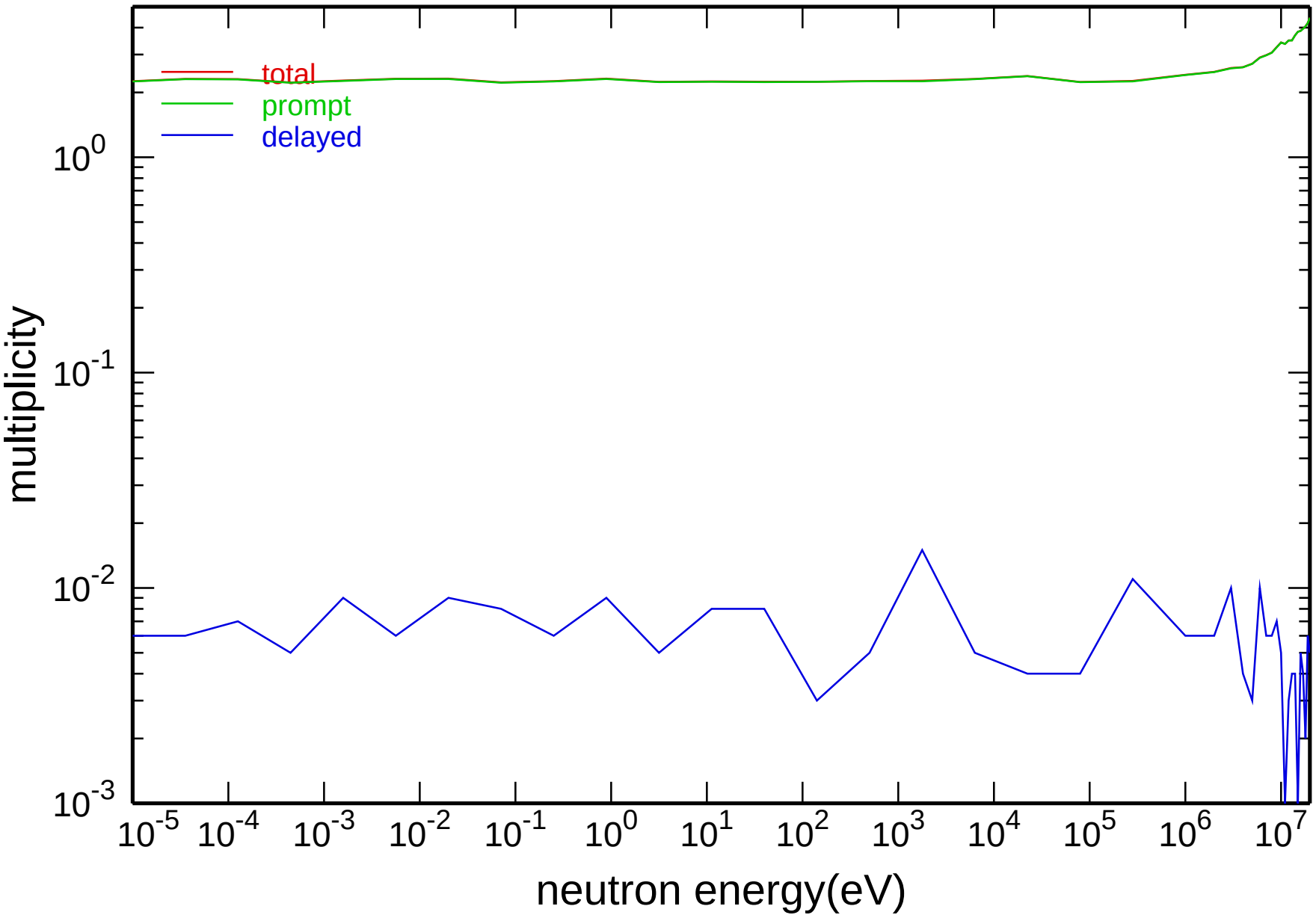
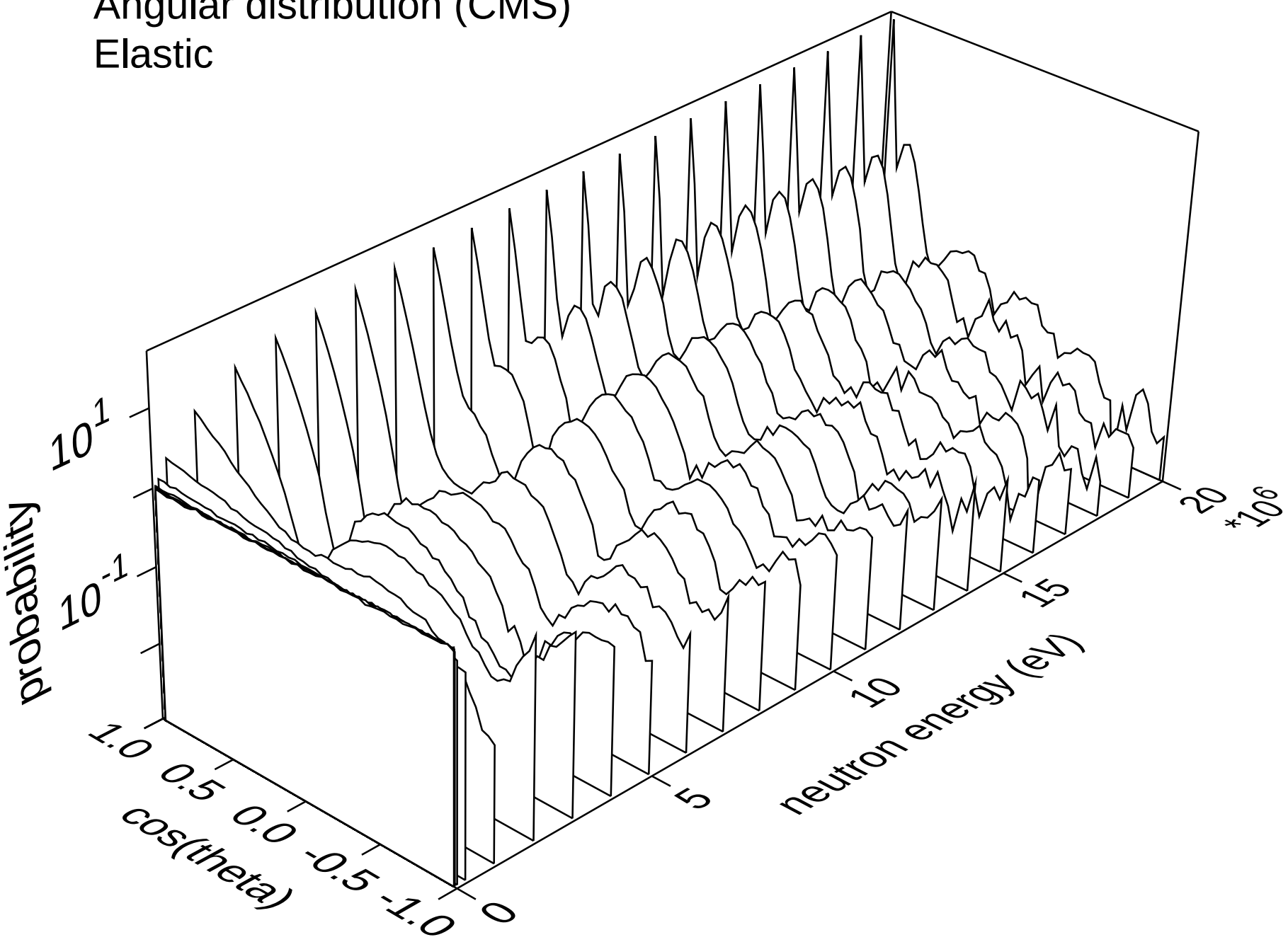
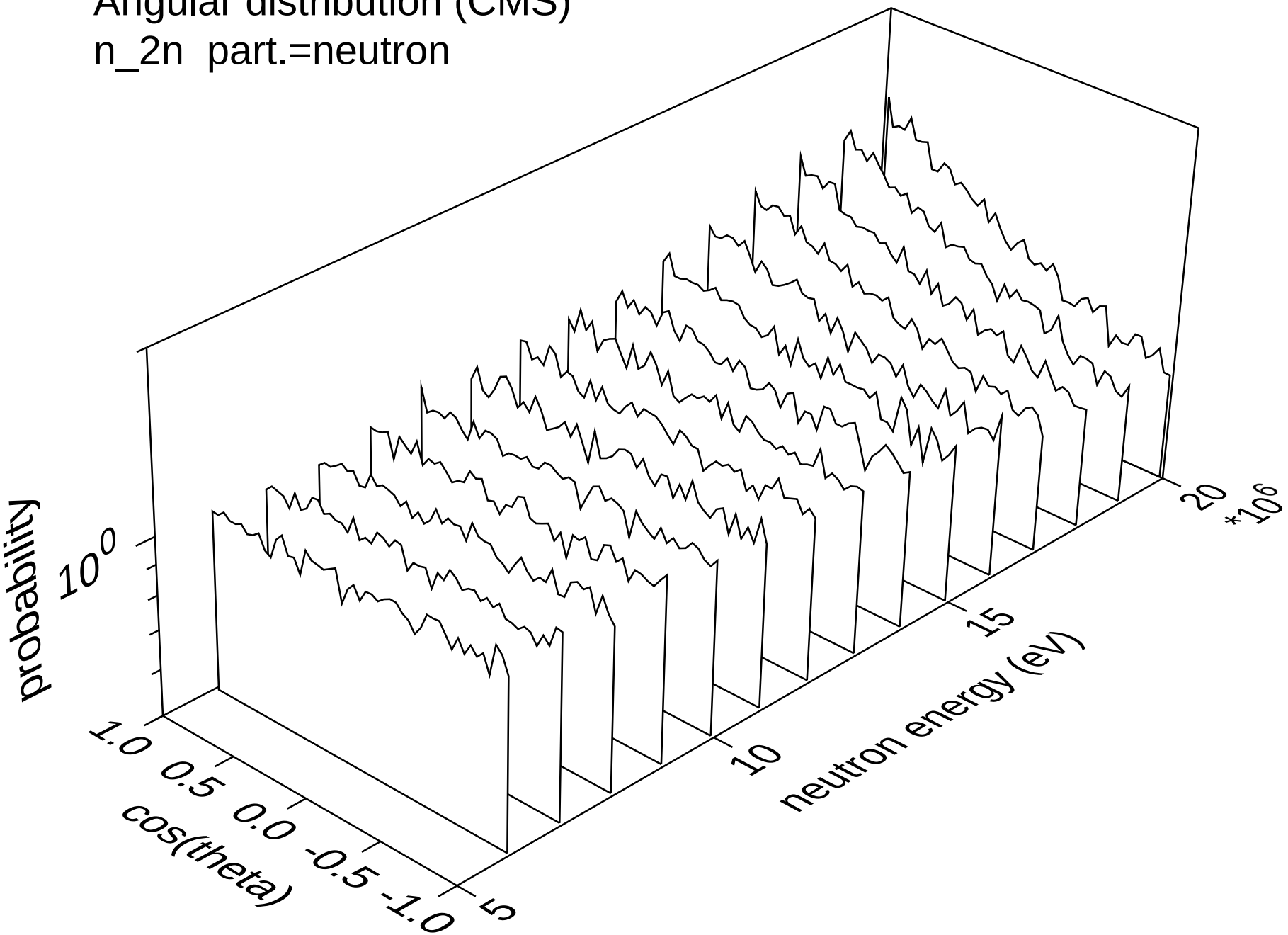
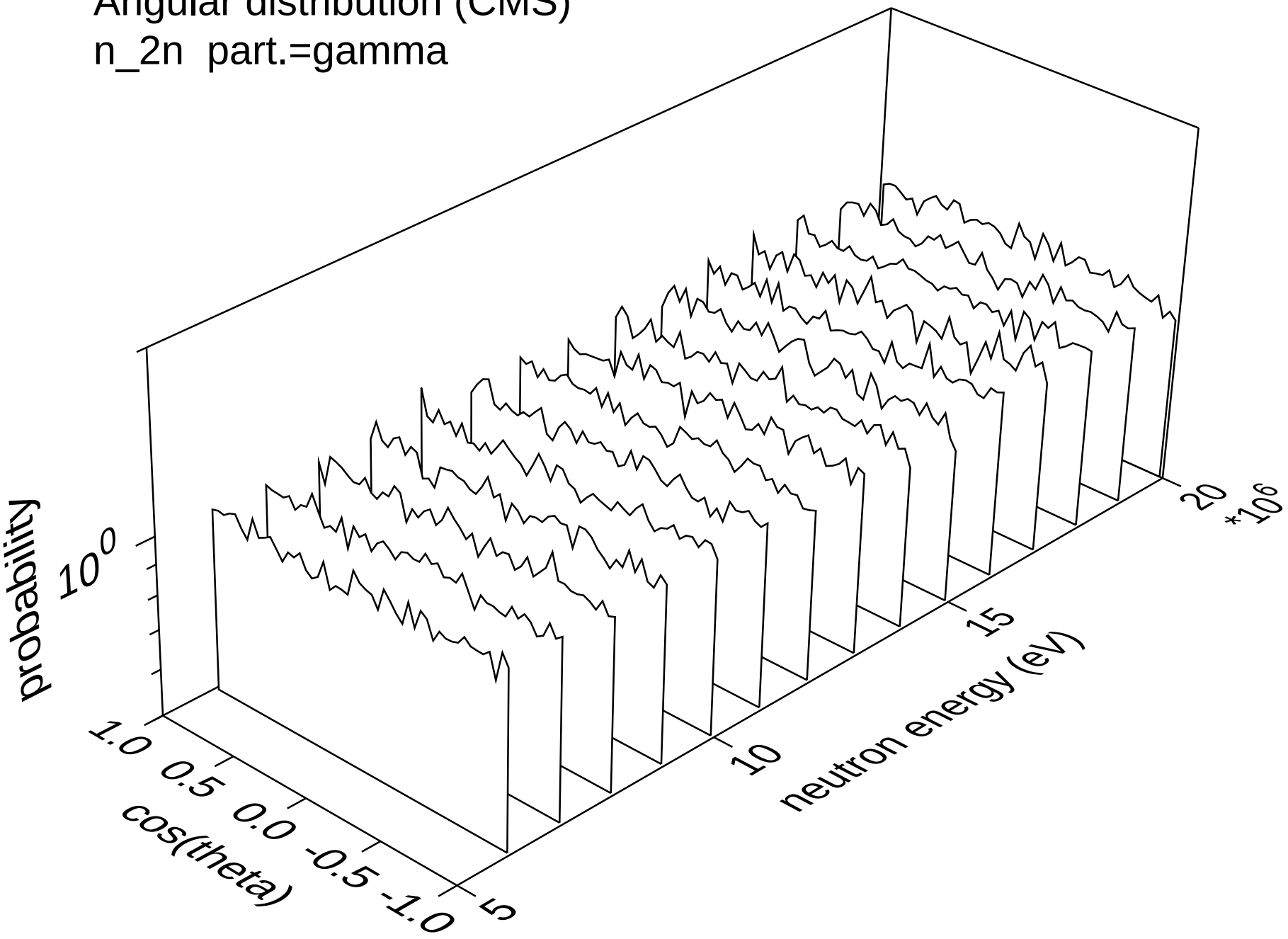

Main Cross Sections
Elastic
Capture
Fission
Inelastic
cross section (barn)
neutron energy(eV)
10^5
10^4
10^3
10^2
10^1
10^0
10^-1
10^-2
10^-3
10^-4
10^-5
10^-4
10^-3
10^-2
10^-1
10^0
10^1
10^2
10^3
10^4
10^5
10^6
10^7

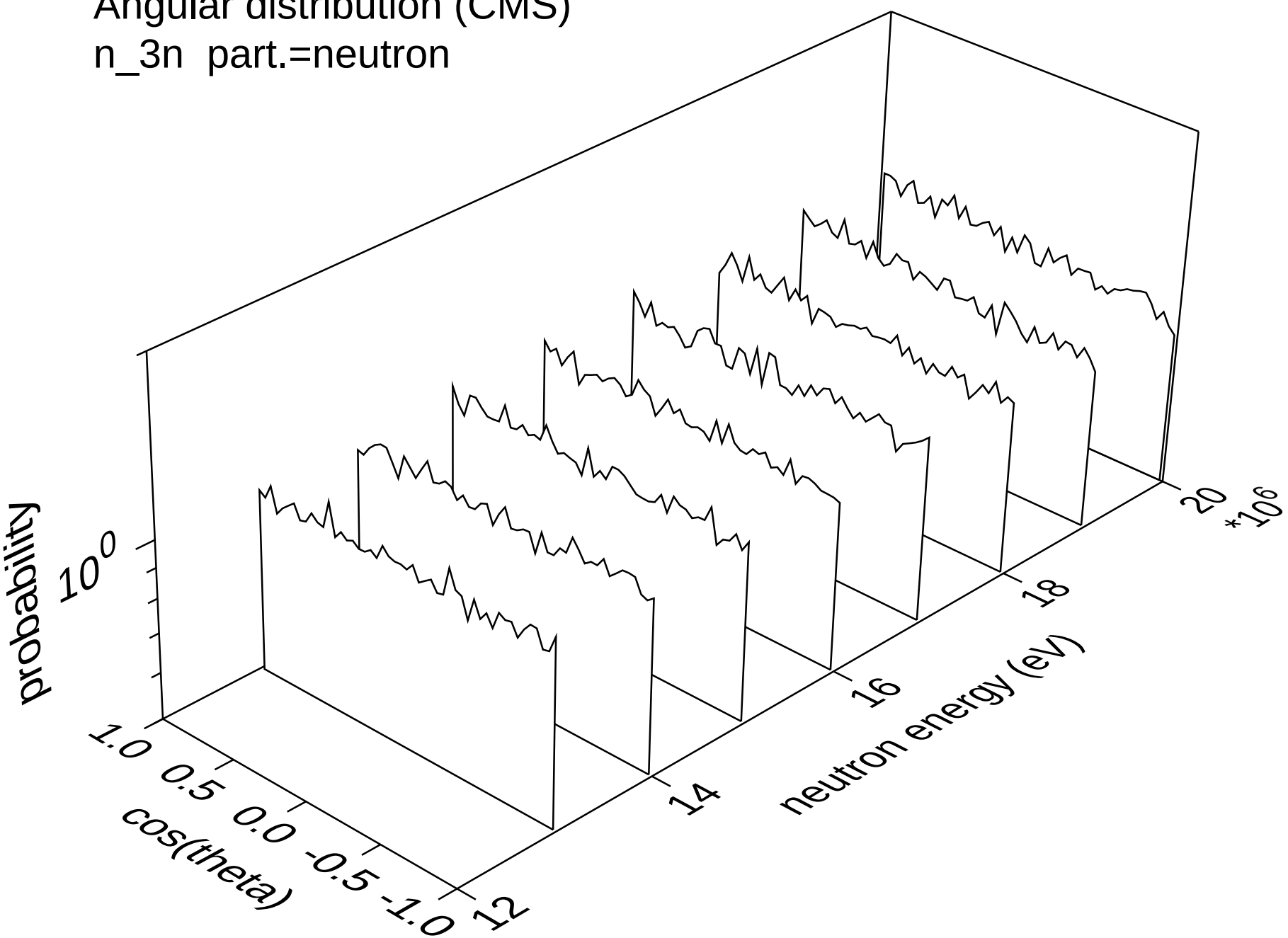

Cross Section
cross section (barn)
neutron energy(eV)
*10^6
n_2n
n_3n
n_4n
n_n_1
n_n_cont
n_n_tot
10^0
10^-2
10^-4
10^-6
10^-8
10^-10
10^-12
10^-14
0 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20

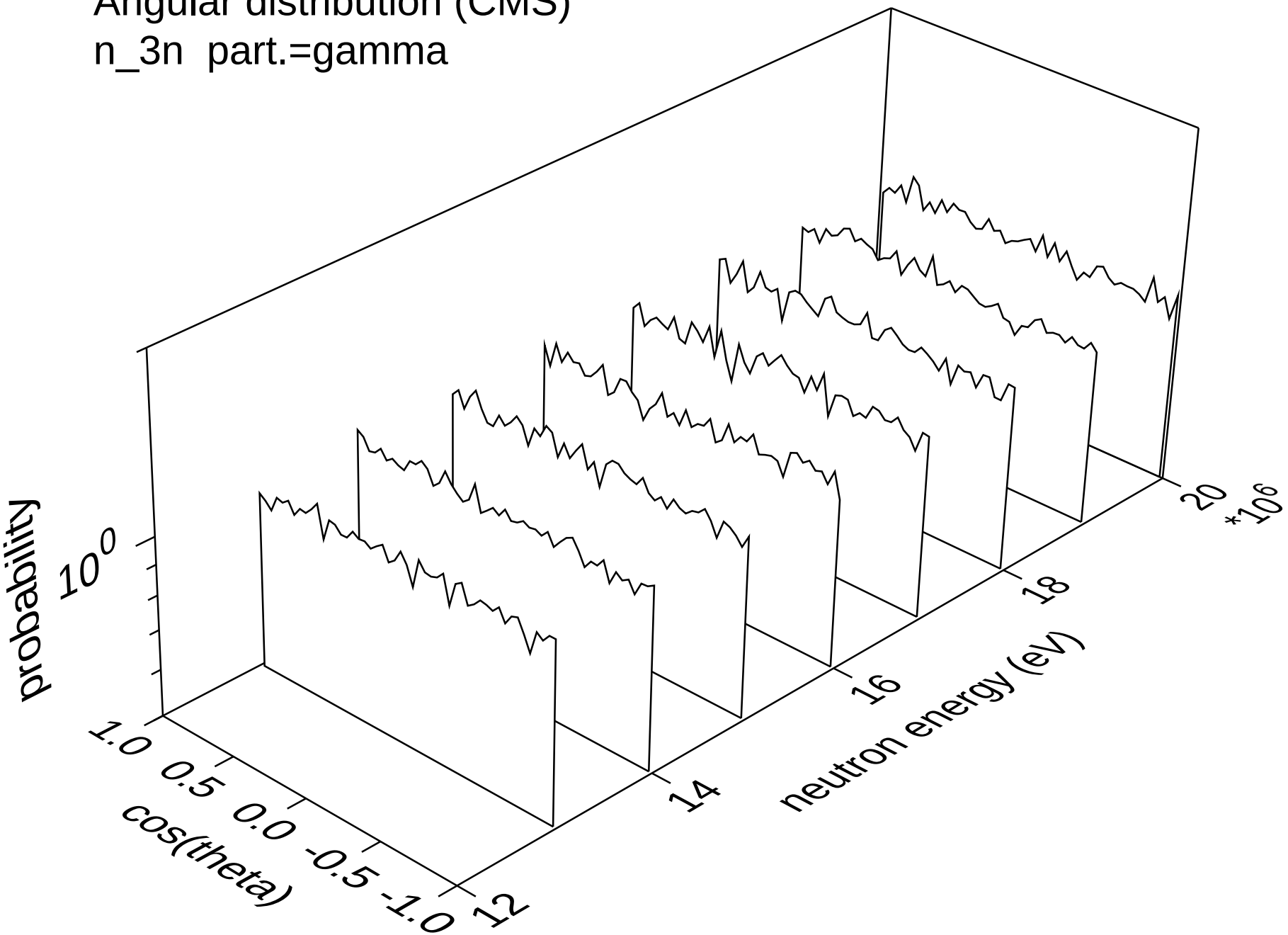

neutron multiplicity for fission
total
prompt
delayed
multiplicity
10^{0}
10^{-1}
10^{-2}
10^{-3}
10^{-5} 10^{-4} 10^{-3} 10^{-2} 10^{-1} 10^{0} 10^{1} 10^{2} 10^{3} 10^{4} 10^{5} 10^{6} 10^{7}
neutron energy(eV)

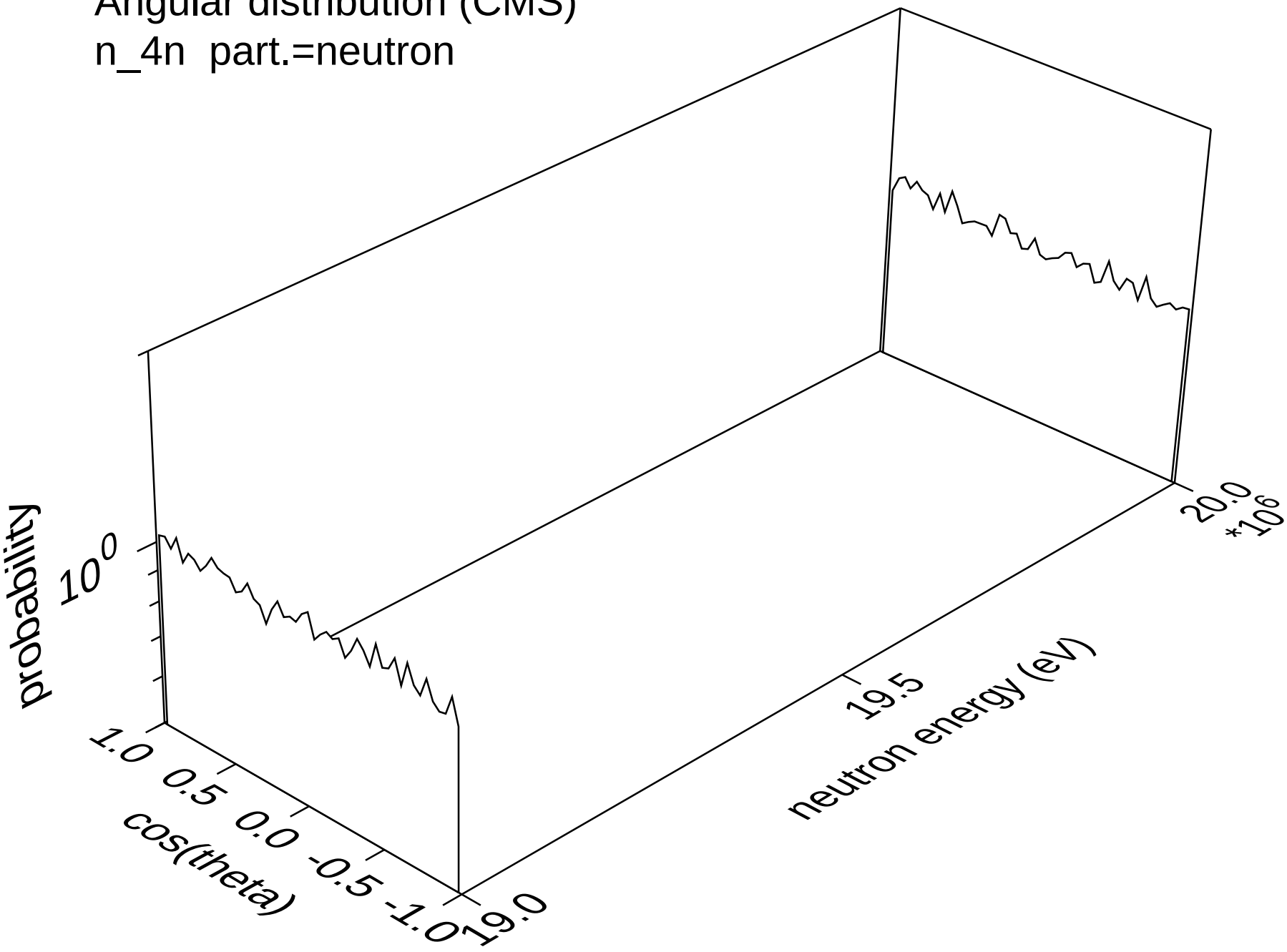

Angular distribution (CMS)
Elastic
probability
10^{1}
10^{-1}
1.0
0.5
0.0
-0.5
-1.0
cos(theta)
0
5
10
15
20
*10^6
neutron energy (eV)

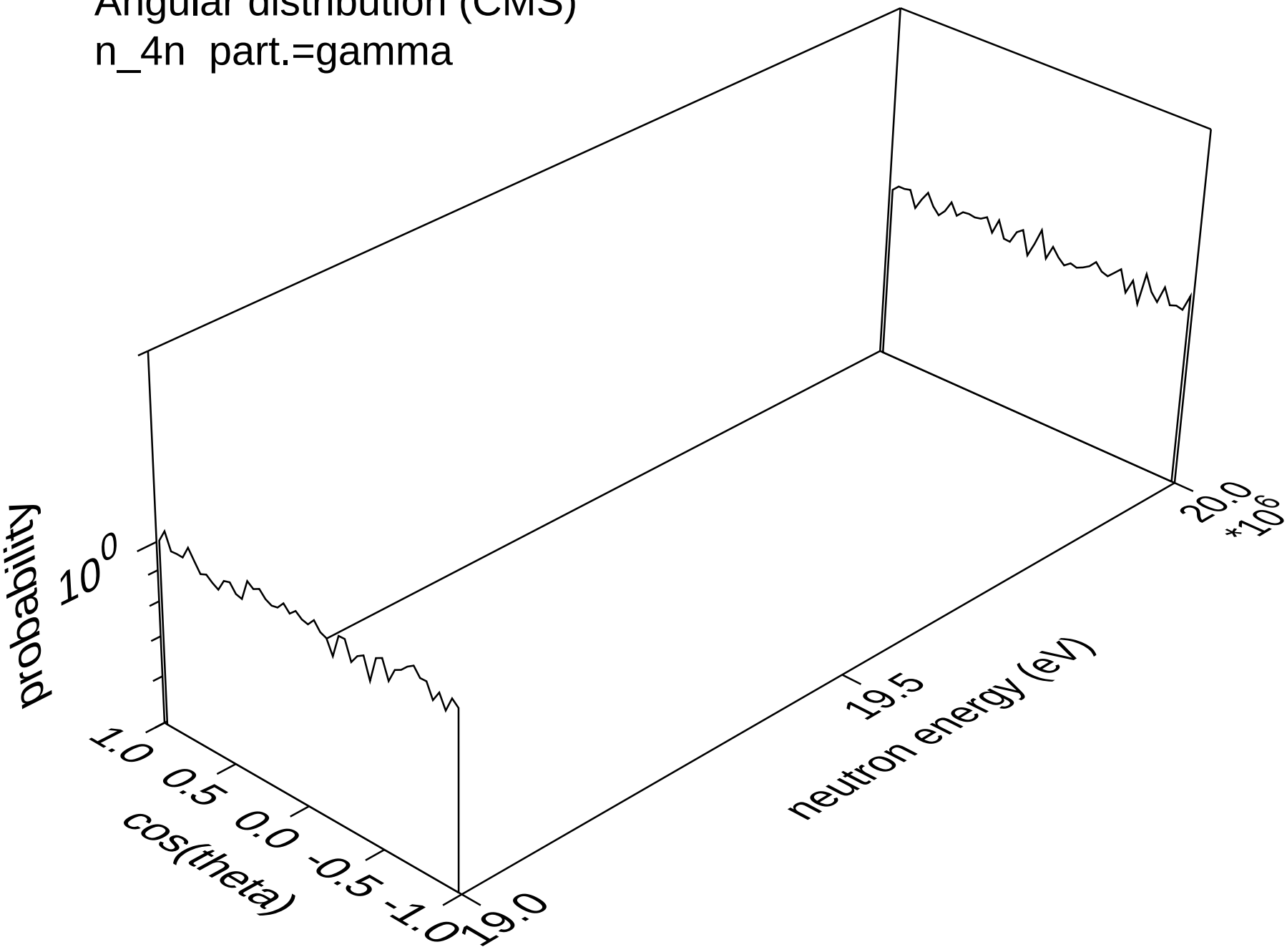

Angular distribution (CMS)
n_2n part.=neutron
probability
10^0
1.0
0.5
0.0
-0.5
-1.0
cos(theta)
5
10
15
20
*10^6
neutron energy (eV)

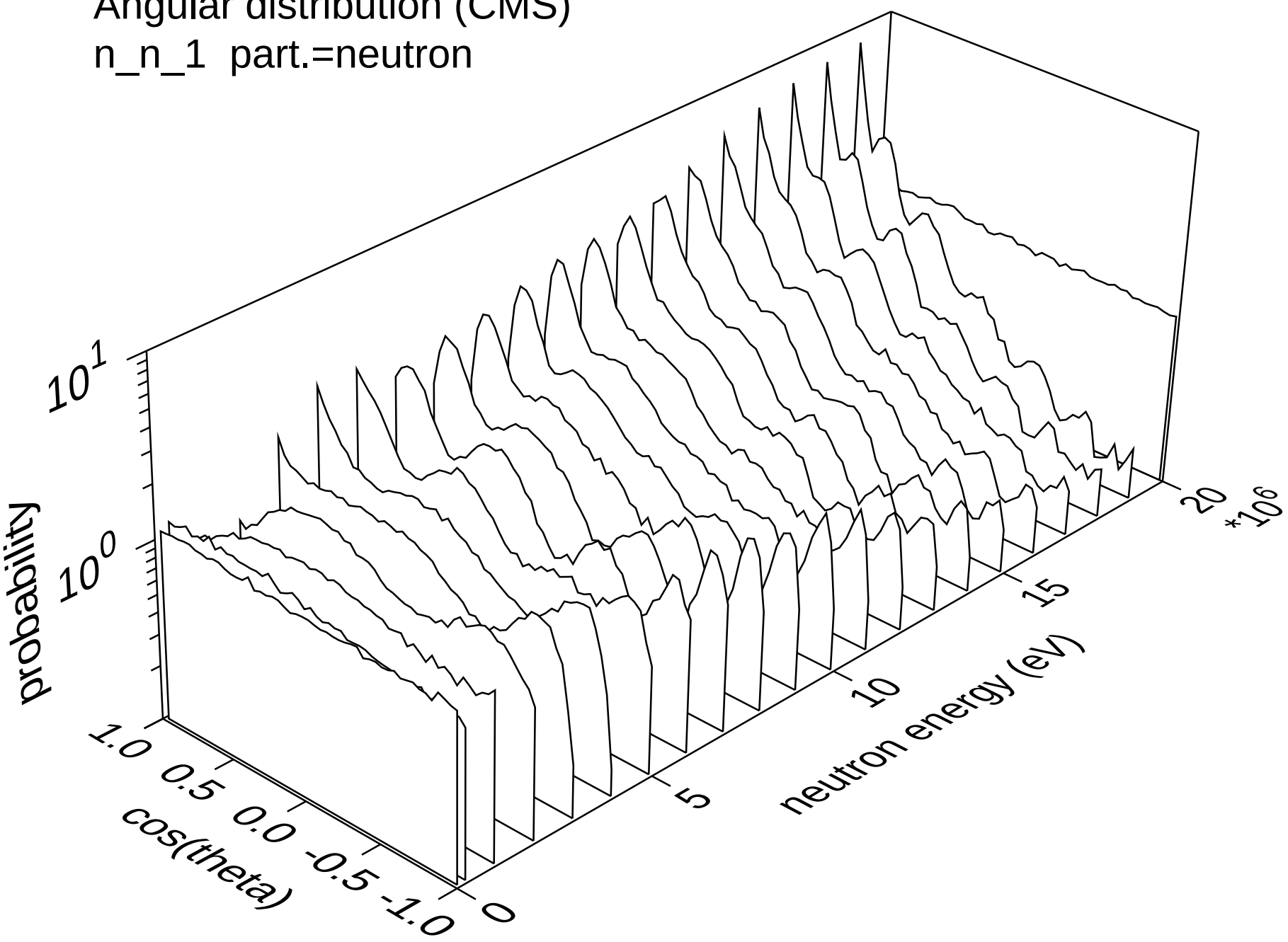

Angular distribution (CMS)
n_2n part.=gamma
probability
10^0
1.0
0.5
0.0
-0.5
-1.0
cos(theta)
5
10
15
20
*10^6
neutron energy (eV)

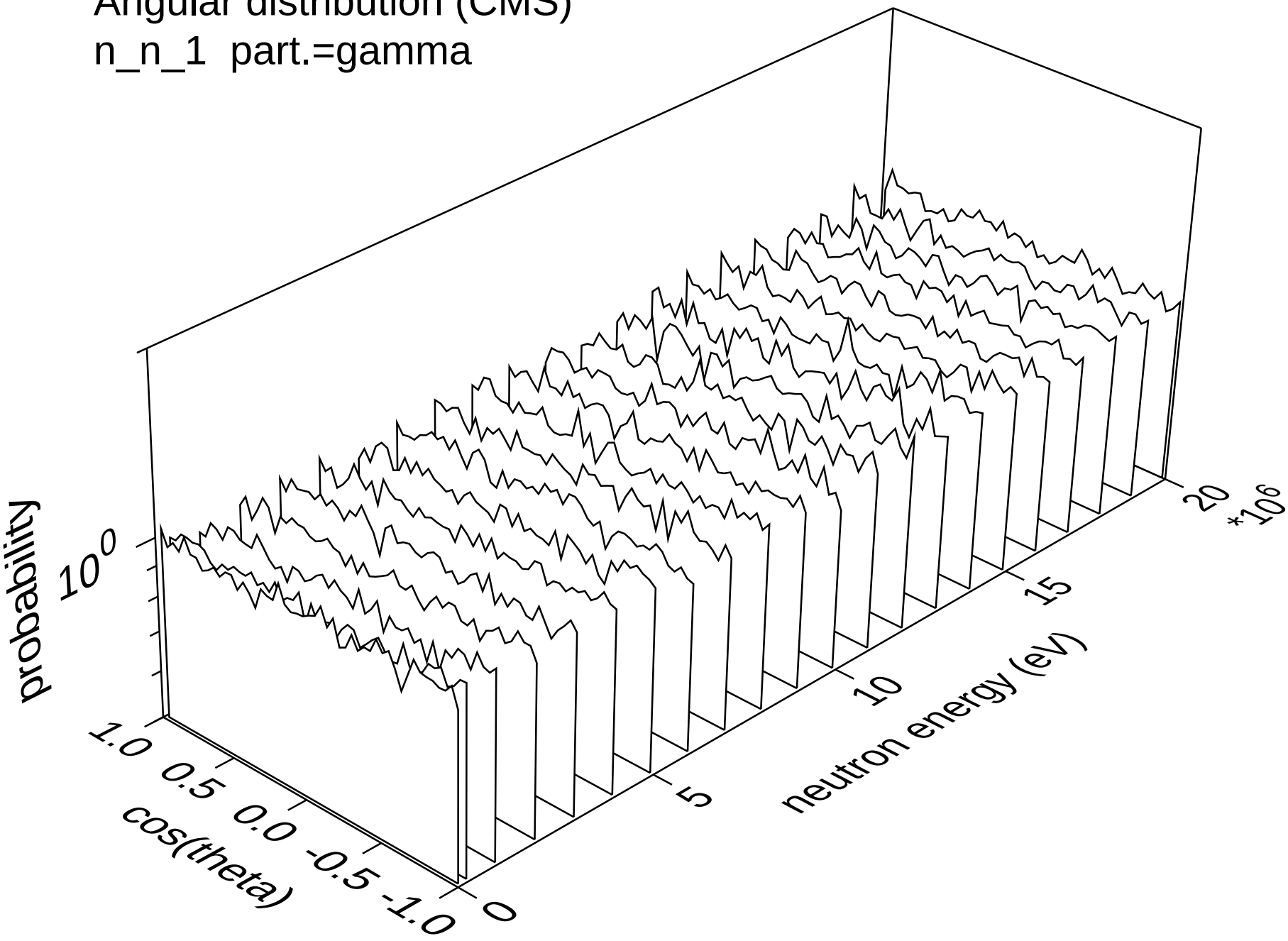

Angular distribution (CMS)
n_3n part.=neutron
probability
10^0
1.0
0.5
0.0
-0.5
-1.0
cos(theta)
12
14
16
18
20
*10^6
neutron energy (eV)

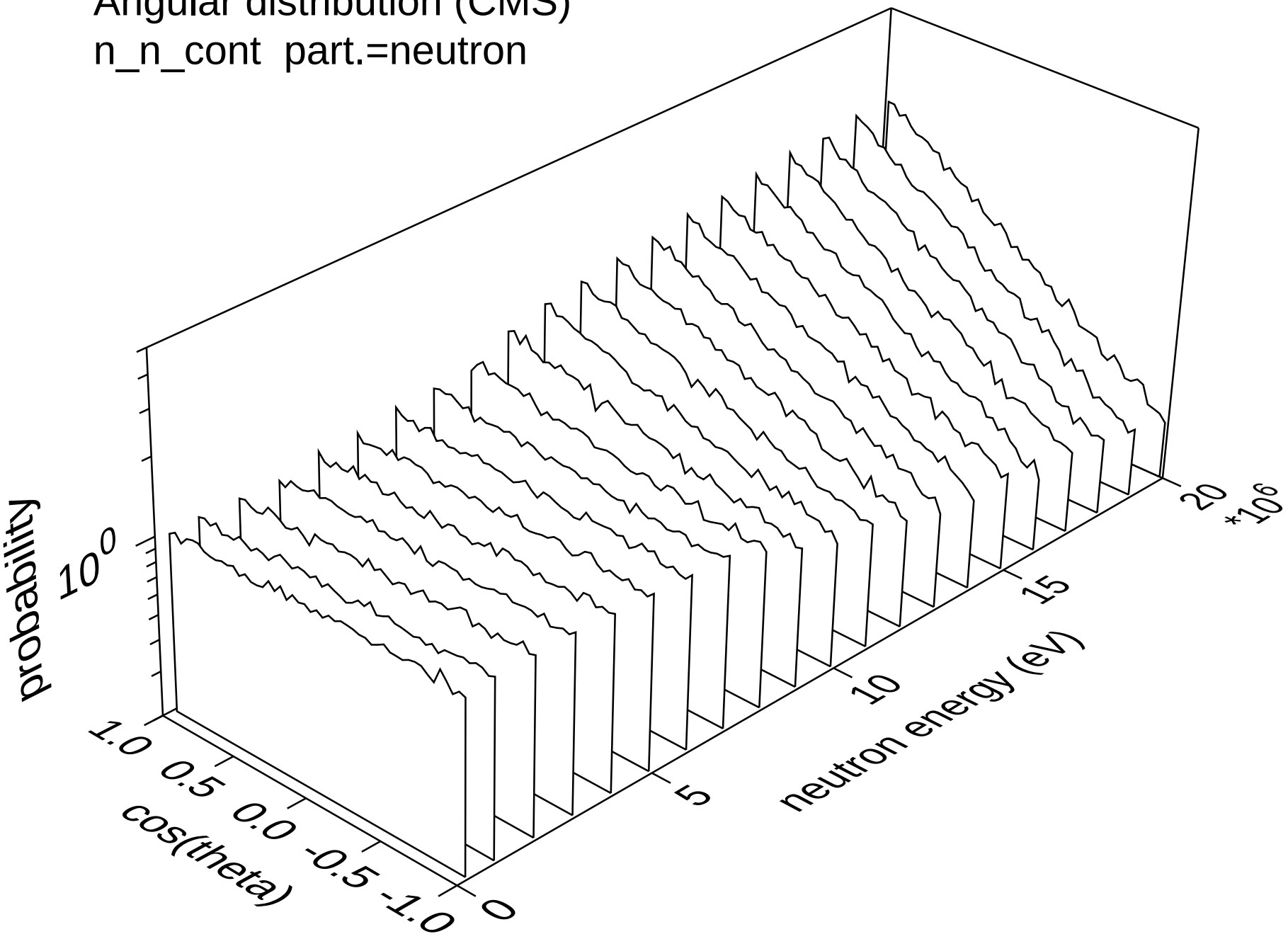

Angular distribution (CMS)
n_3n part.=gamma
probability
10^0
1.0
0.5
0.0
-0.5
-1.0
cos(theta)
12
14
16
18
20
*10^6
neutron energy (eV)

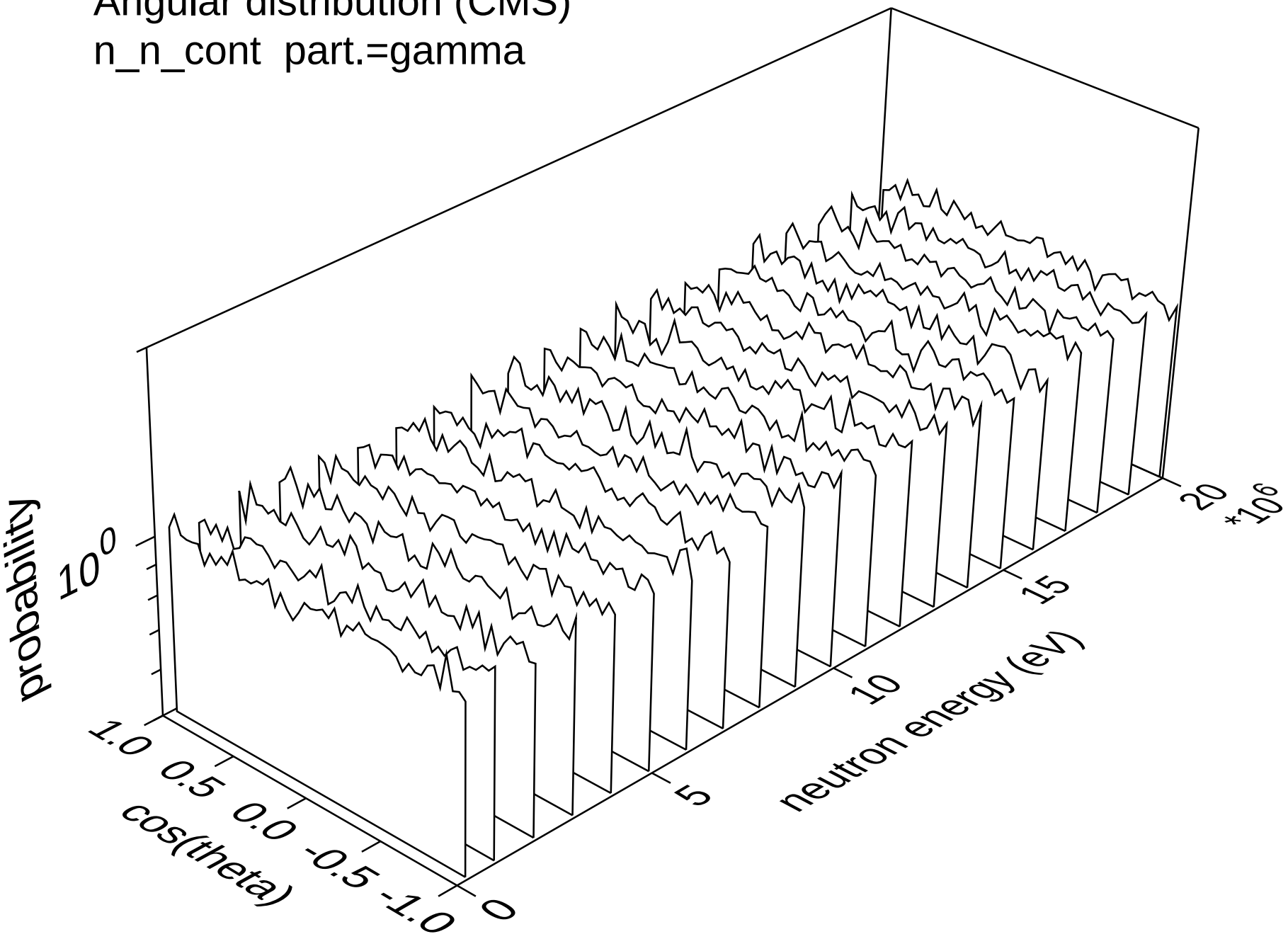

Angular distribution (CMS)
n_4n part.=neutron
probability
10^0
1.0
0.5
0.0
-0.5
-1.0
cos(theta)
19.0
19.5
20.0
*10^6
neutron energy (eV)

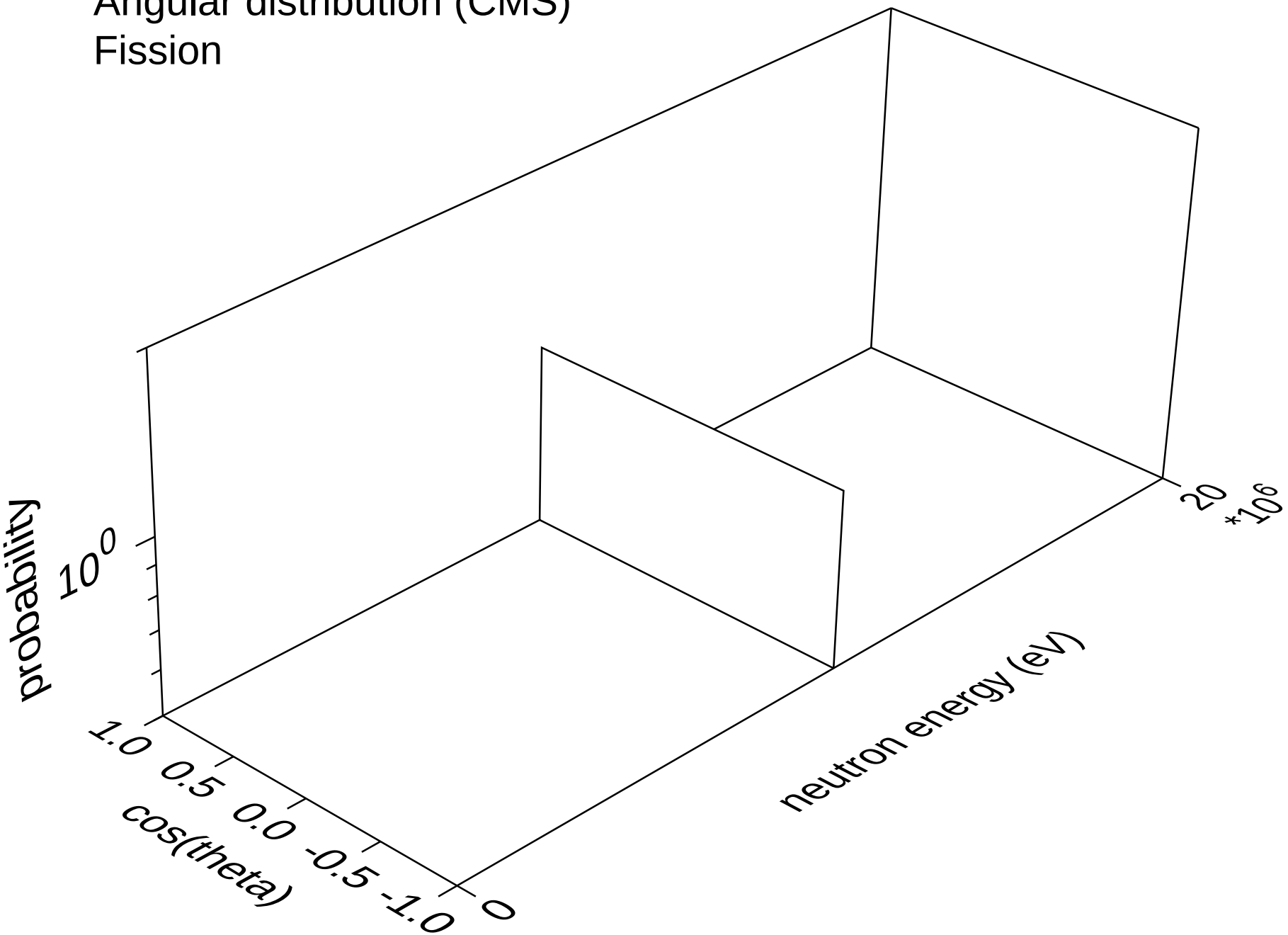

Angular distribution (CMS)
n_4n part.=gamma
probability
10^0
1.0
0.5
0.0
-0.5
-1.0
cos(theta)
19.0
19.5
20.0
*10^6
neutron energy (eV)

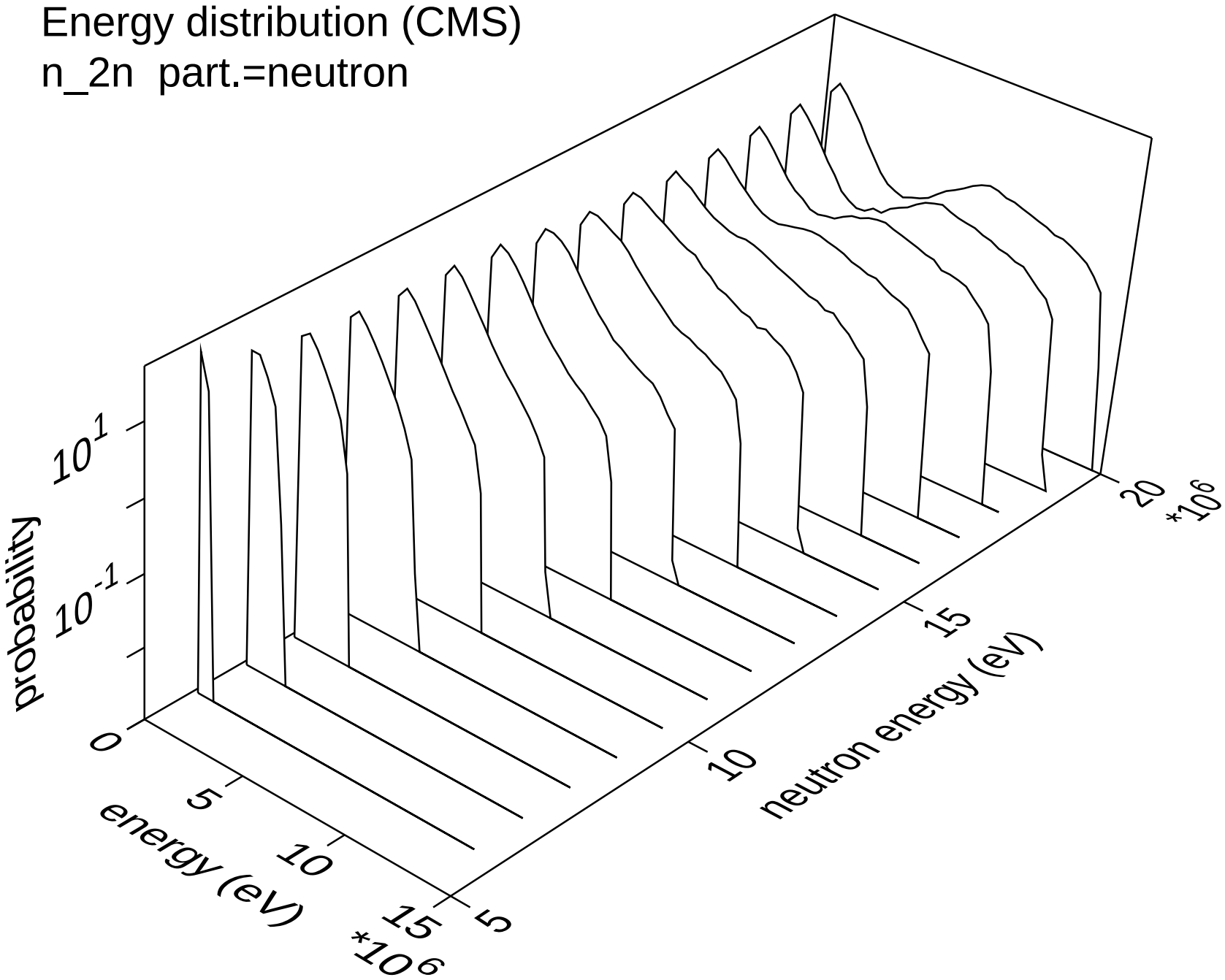

Angular distribution (CMS)
n_n_1 part.=neutron
probability
10^1
10^0
1.0
0.5
0.0
-0.5
-1.0
cos(theta)
0
5
10
15
20
*10^6
neutron energy (eV)

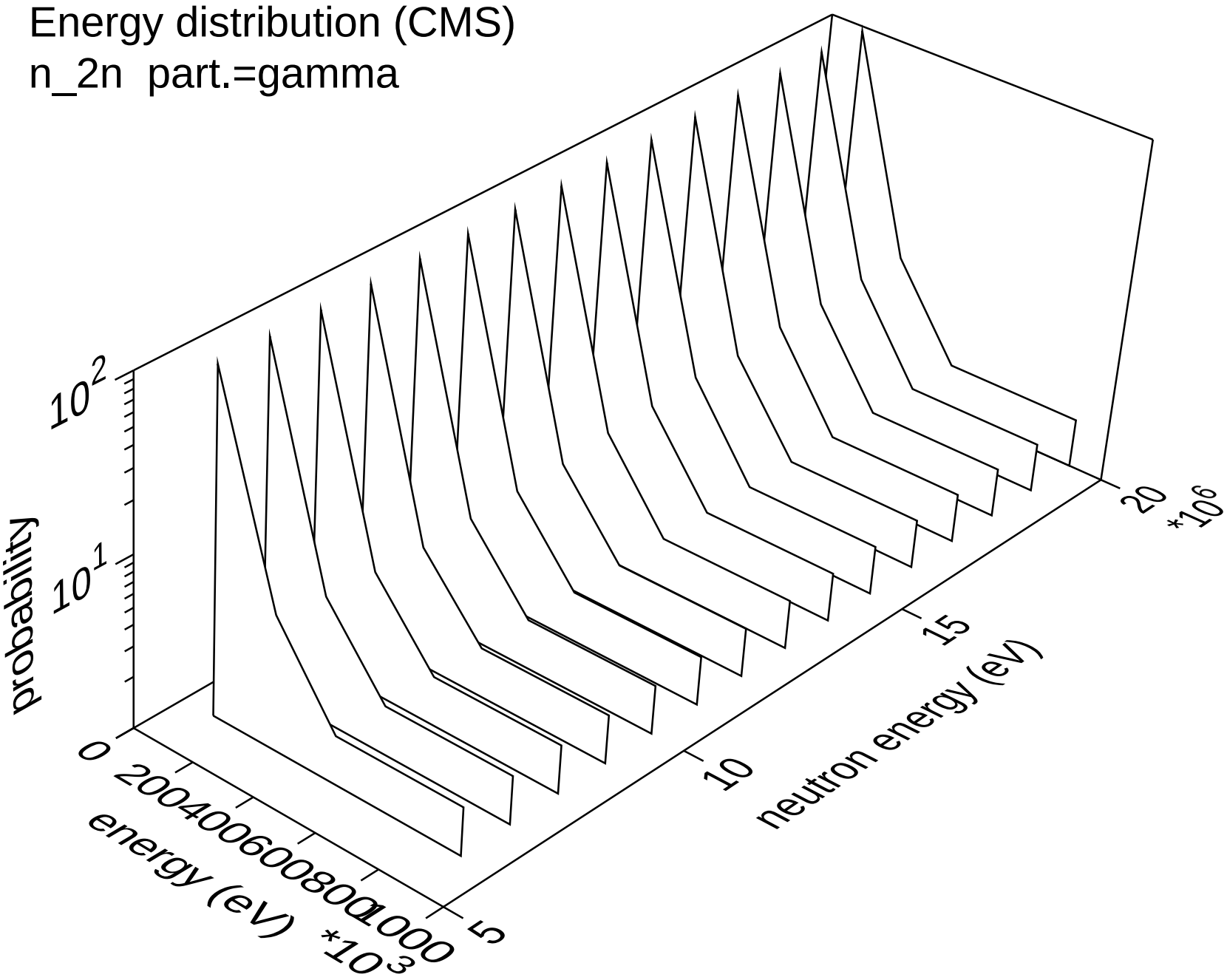

Angular distribution (CMS)
n_n_1 part.=gamma
probability
10^0
1.0
0.5
0.0
-0.5
-1.0
cos(theta)
0
5
10
15
20
*10^6
neutron energy (eV)

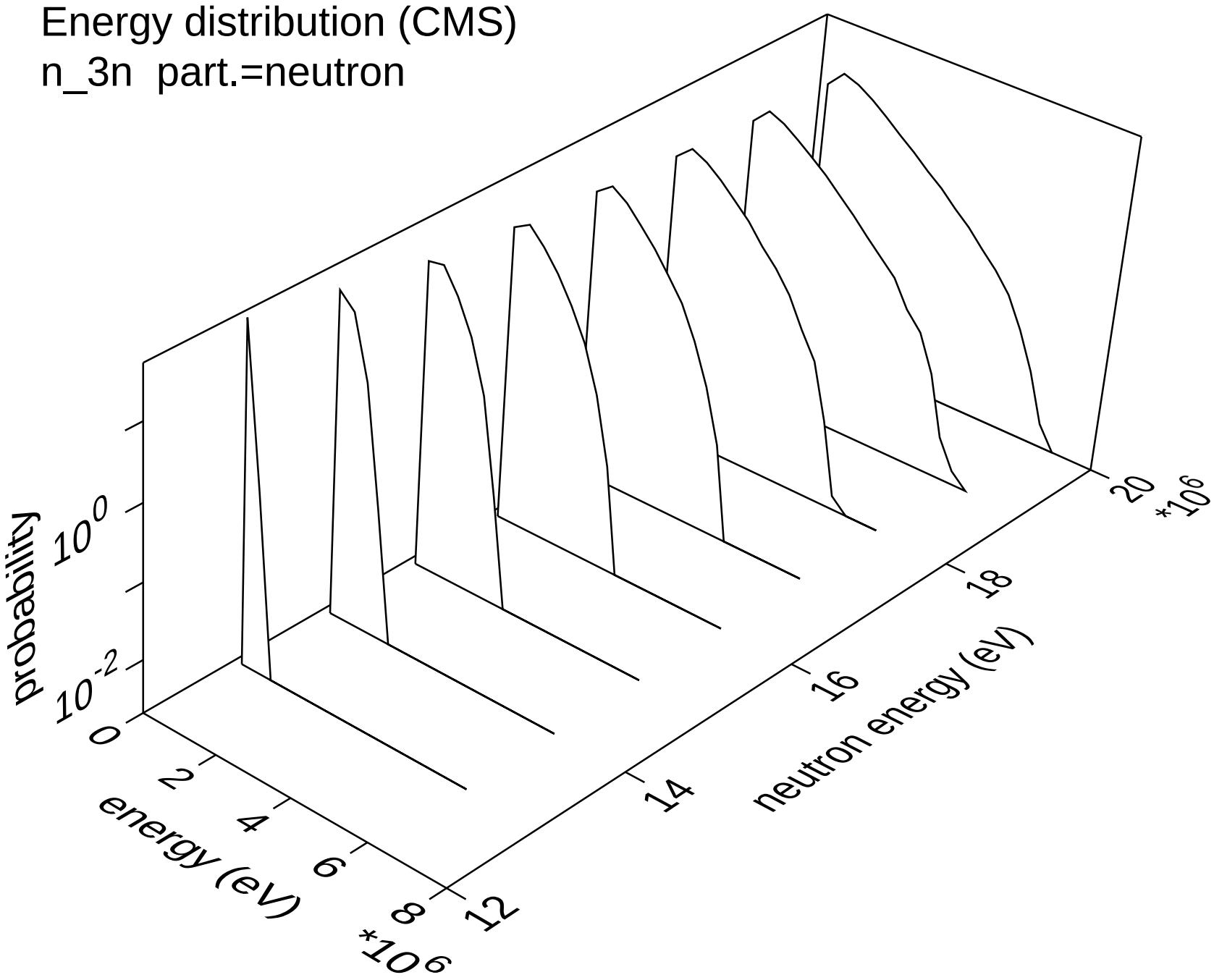

Angular distribution (CMS)
n_n_cont part.=neutron
probability
10^0
1.0
0.5
0.0
-0.5
-1.0
cos(theta)
0
5
10
15
20
*10^6
neutron energy (eV)

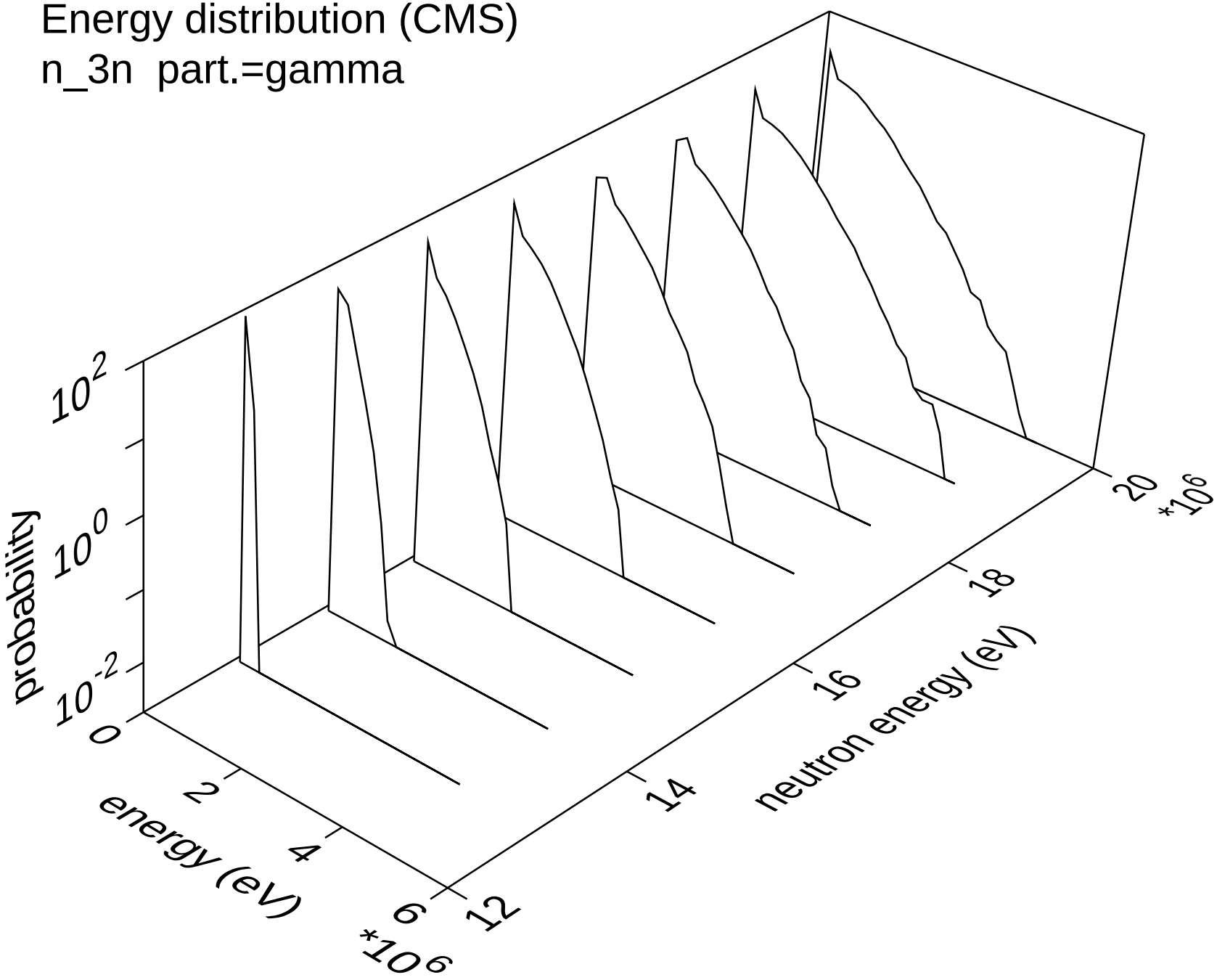

Angular distribution (CMS)
n_n_cont part.=gamma
probability
10^0
1.0
0.5
0.0
-0.5
-1.0
cos(theta)
0
5
10
15
20
*10^6
neutron energy (eV)

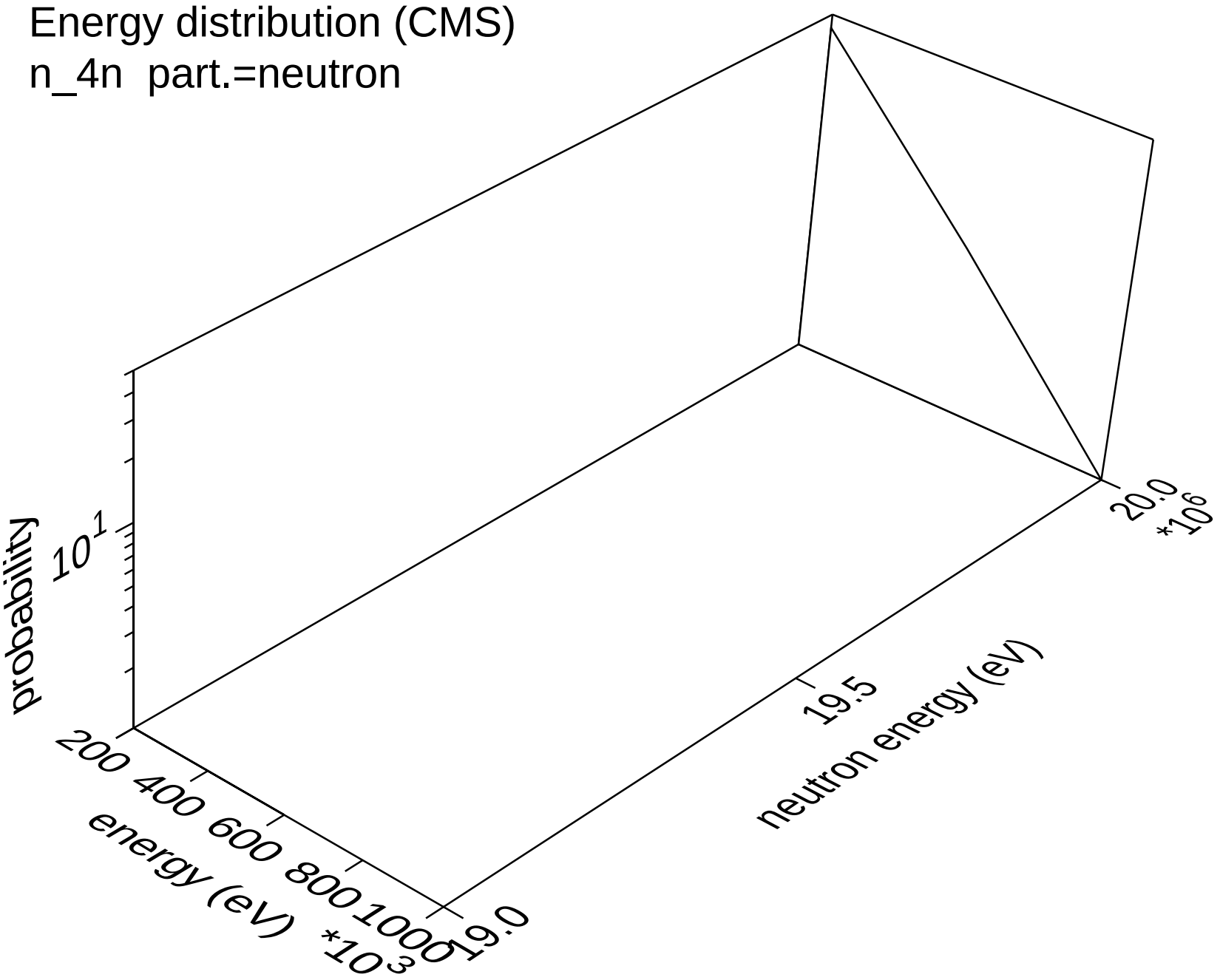

Angular distribution (CMS)
Fission
probability
10^0
1.0
0.5
0.0
-0.5
-1.0
cos(theta)
0
20
*10^6
neutron energy (eV)

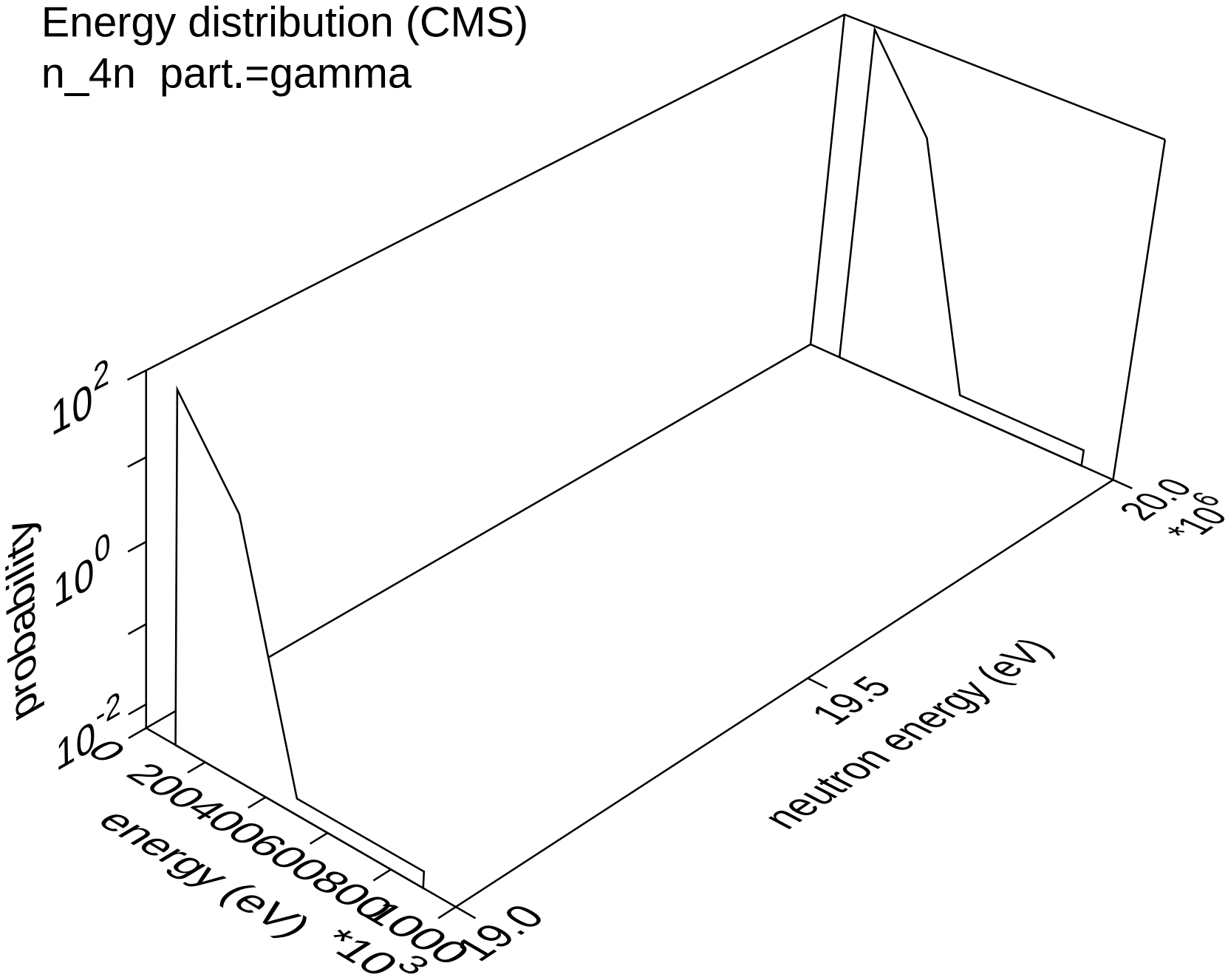

Energy distribution (CMS)
n_2n part.=neutron
probability
10^{1}
10^{-1}
0
5
10
15
*10^6
energy (eV)
5
10
15
20
*10^6
neutron energy (eV)

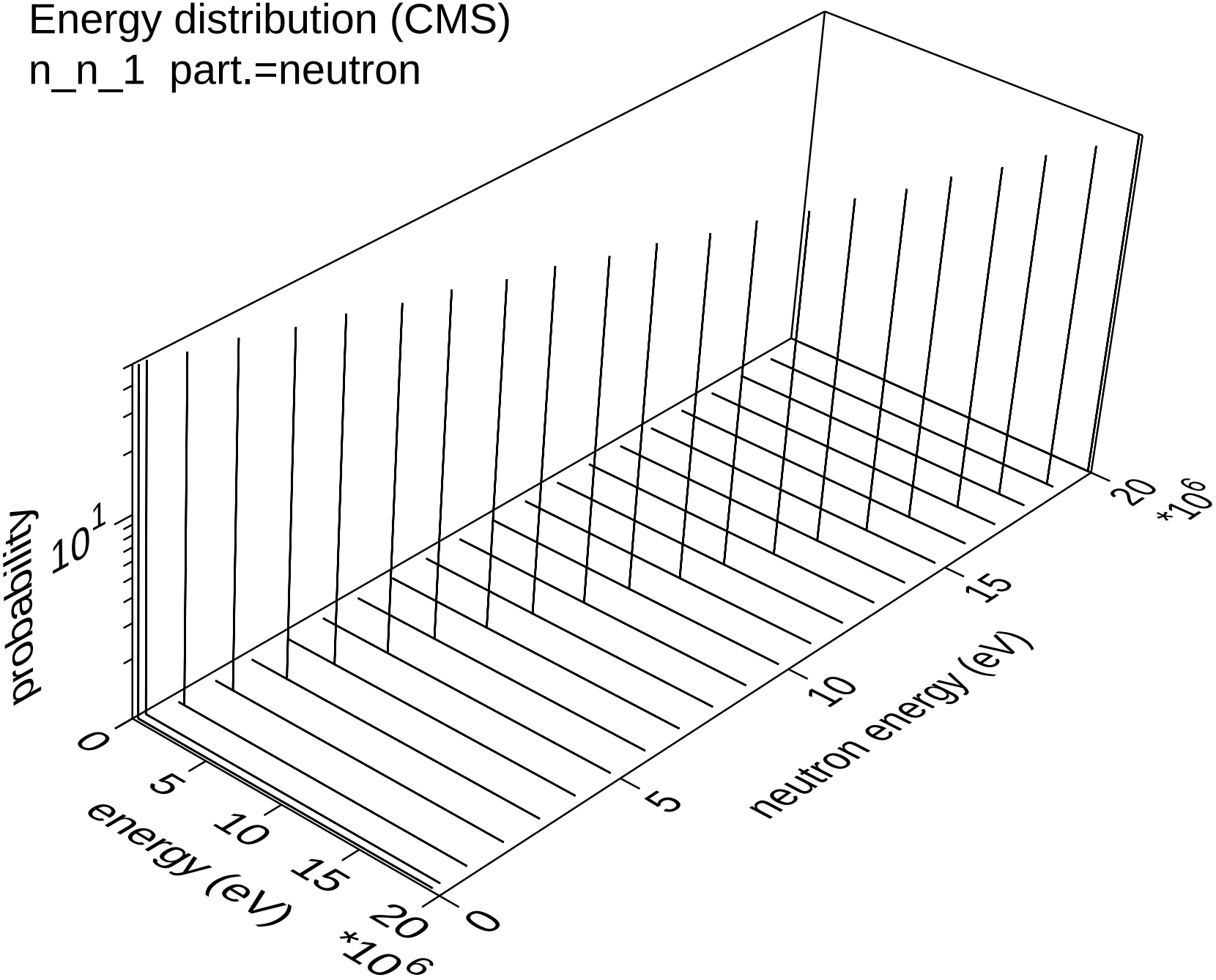

Energy distribution (CMS)
n_2n part.=gamma
probability
10^2
10^1
0
200
400
600
800
1000
energy (eV) *10^3
5
10
15
20
neutron energy (eV) *10^6

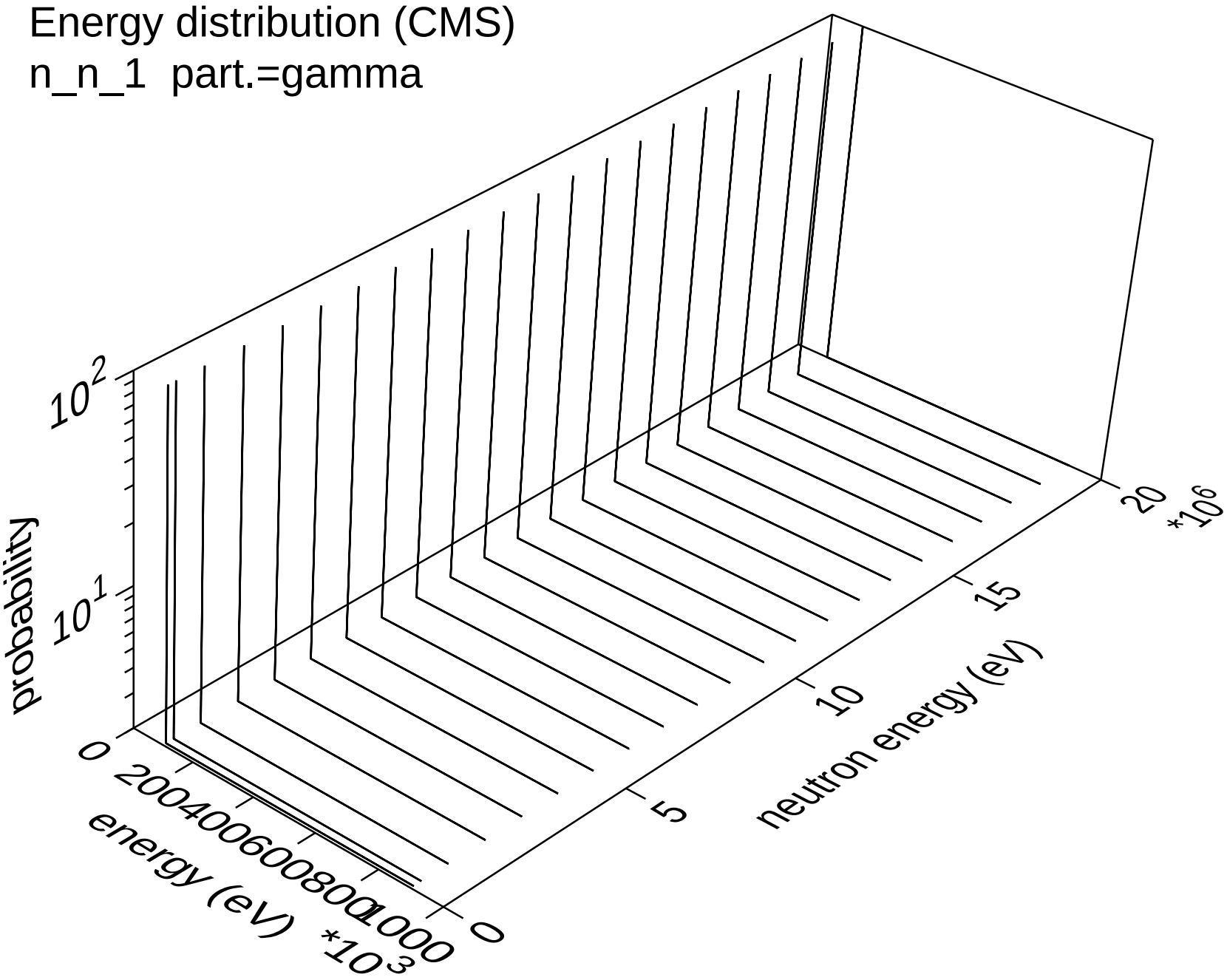

Energy distribution (CMS)
n_3n part.=neutron
probability
10^0
10^{-2}
0
2
4
6
8
*10 6
energy (eV)
12
14
16
18
20
*10^6
neutron energy (eV)

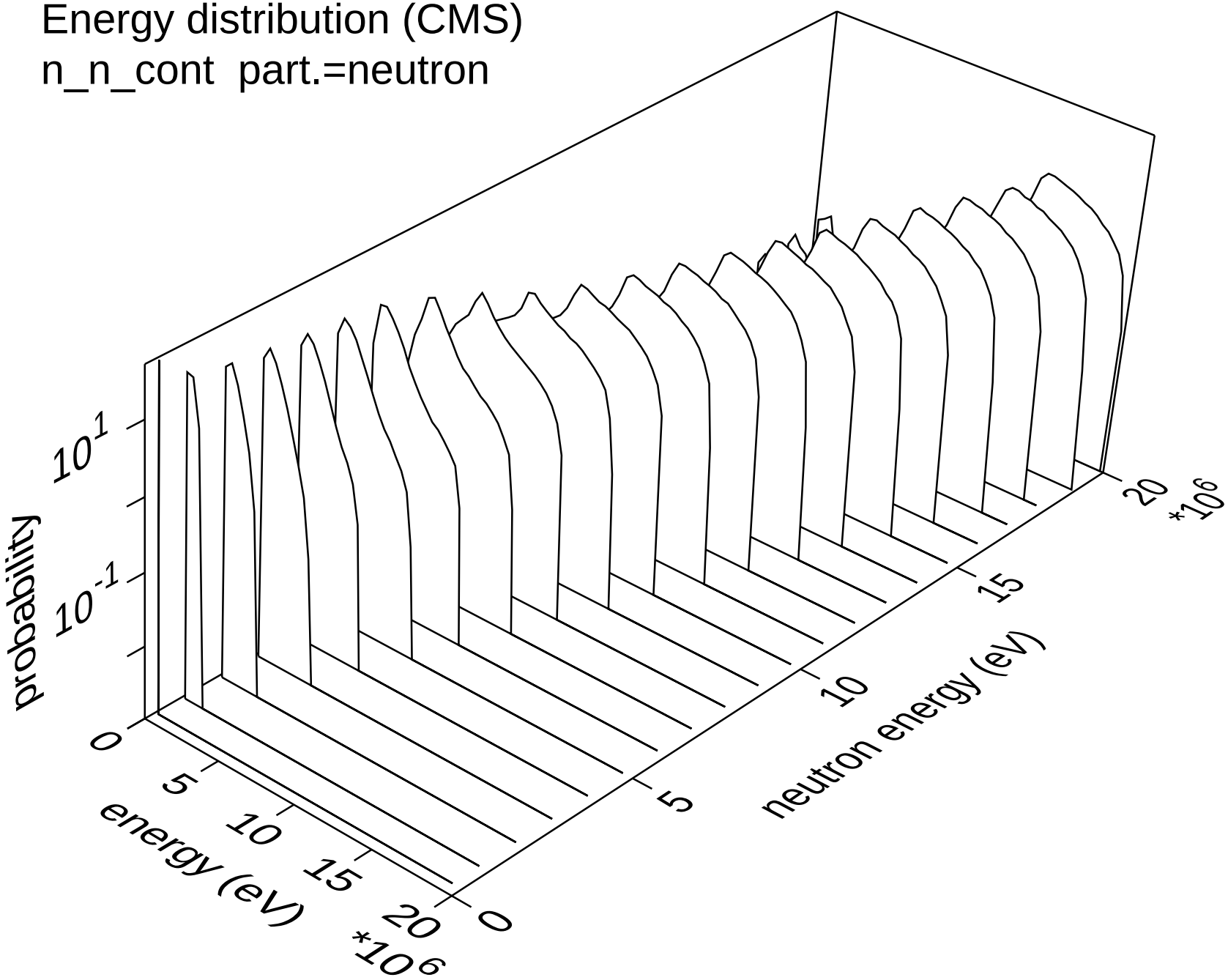

Energy distribution (CMS)
n_3n part.=gamma
probability
10^{2}
10^{0}
10^{-2}
0
2
4
6
*10^6
energy (eV)
12
14
16
18
20
*10^6
neutron energy (eV)

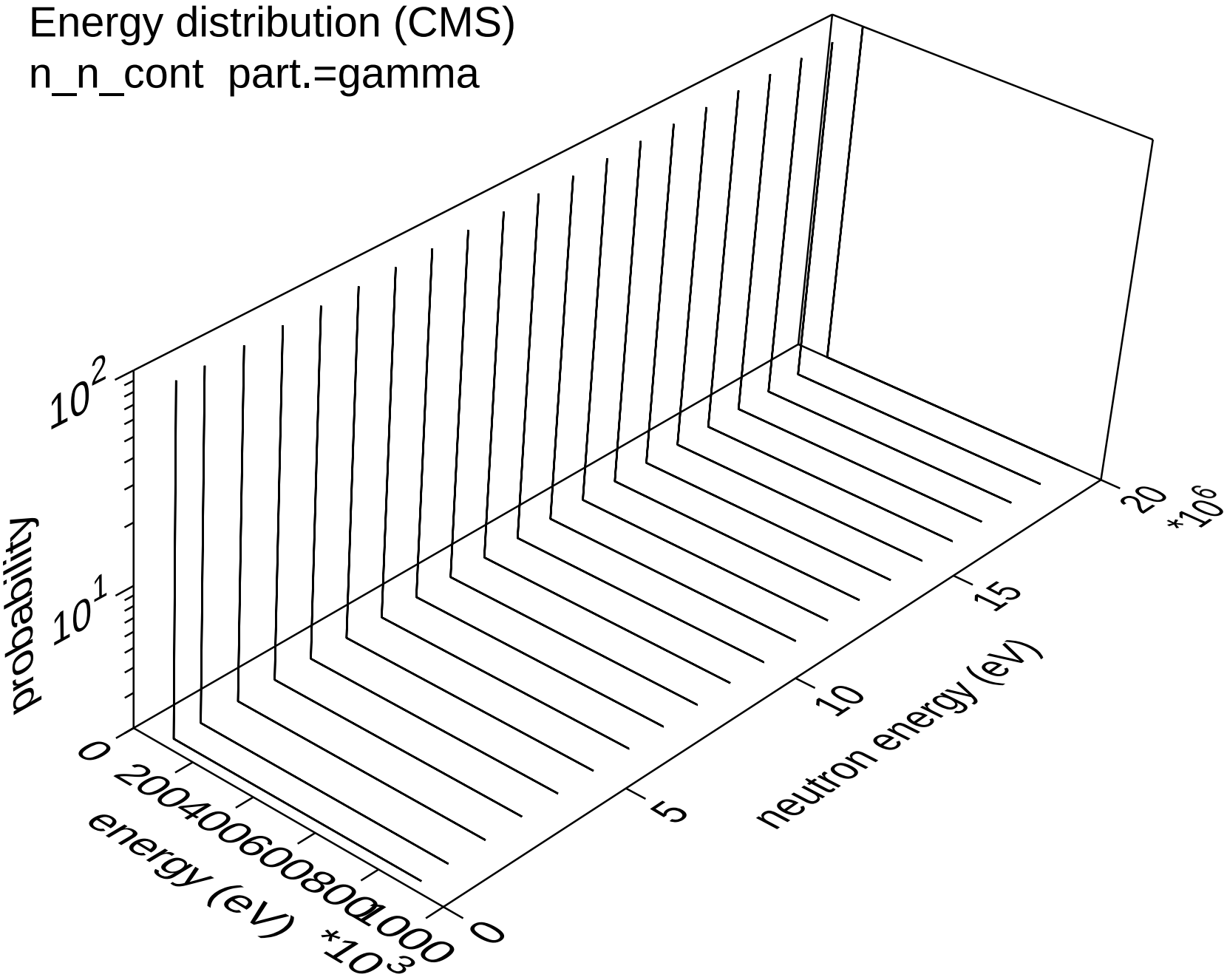

Energy distribution (CMS)
n_4n part.=neutron
probability
10^{1}
200 400 600 800 1000
energy (eV) *10^{3}
19.0
19.5
20.0
*10^{6}
neutron energy (eV)

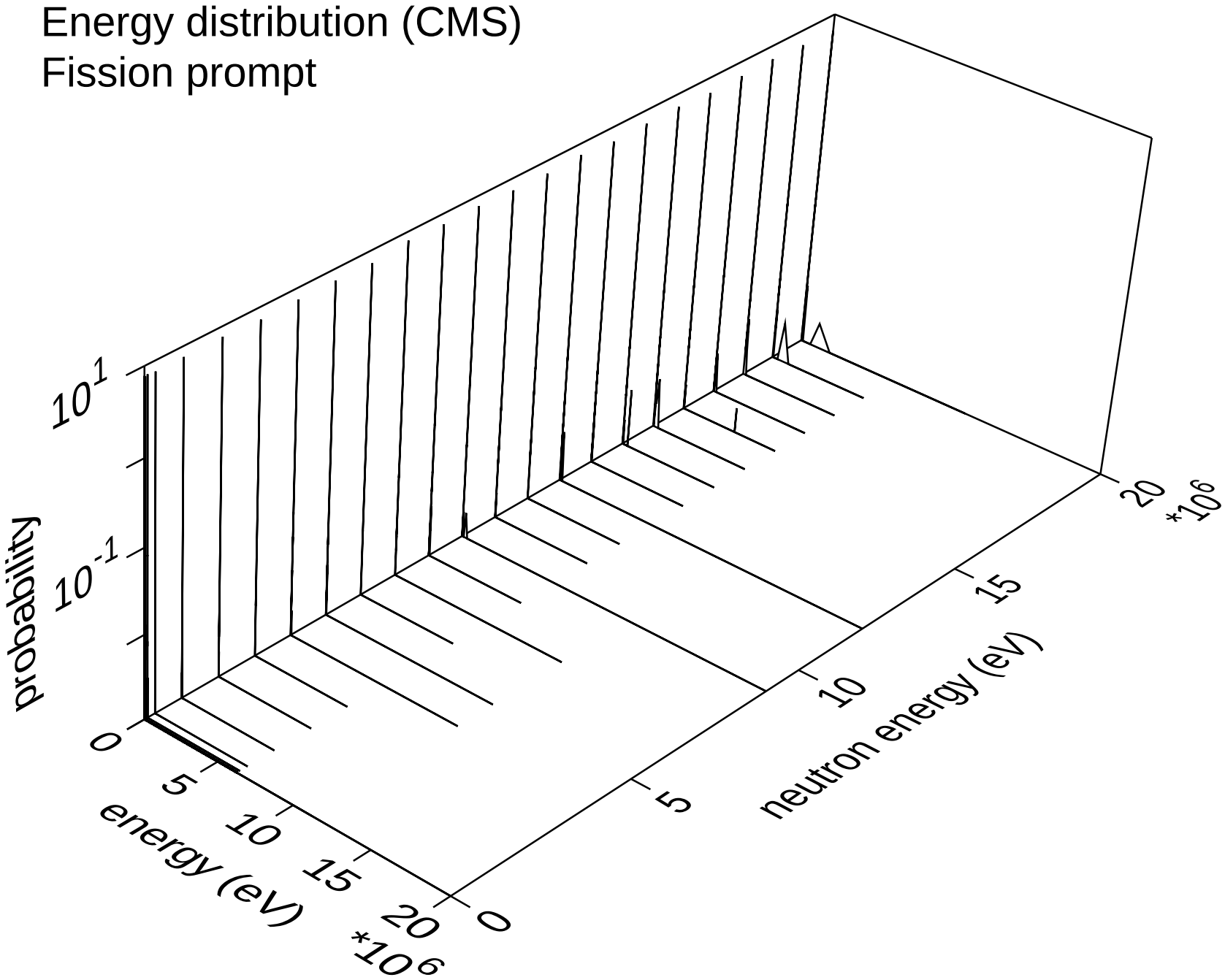

Energy distribution (CMS)
n_4n part.=gamma
probability
10^{2}
10^{0}
10^{-2}
0
200
400
600
800
1000
energy (eV) *10^{3}
19.0
19.5
20.0
*10^{6}
neutron energy (eV)

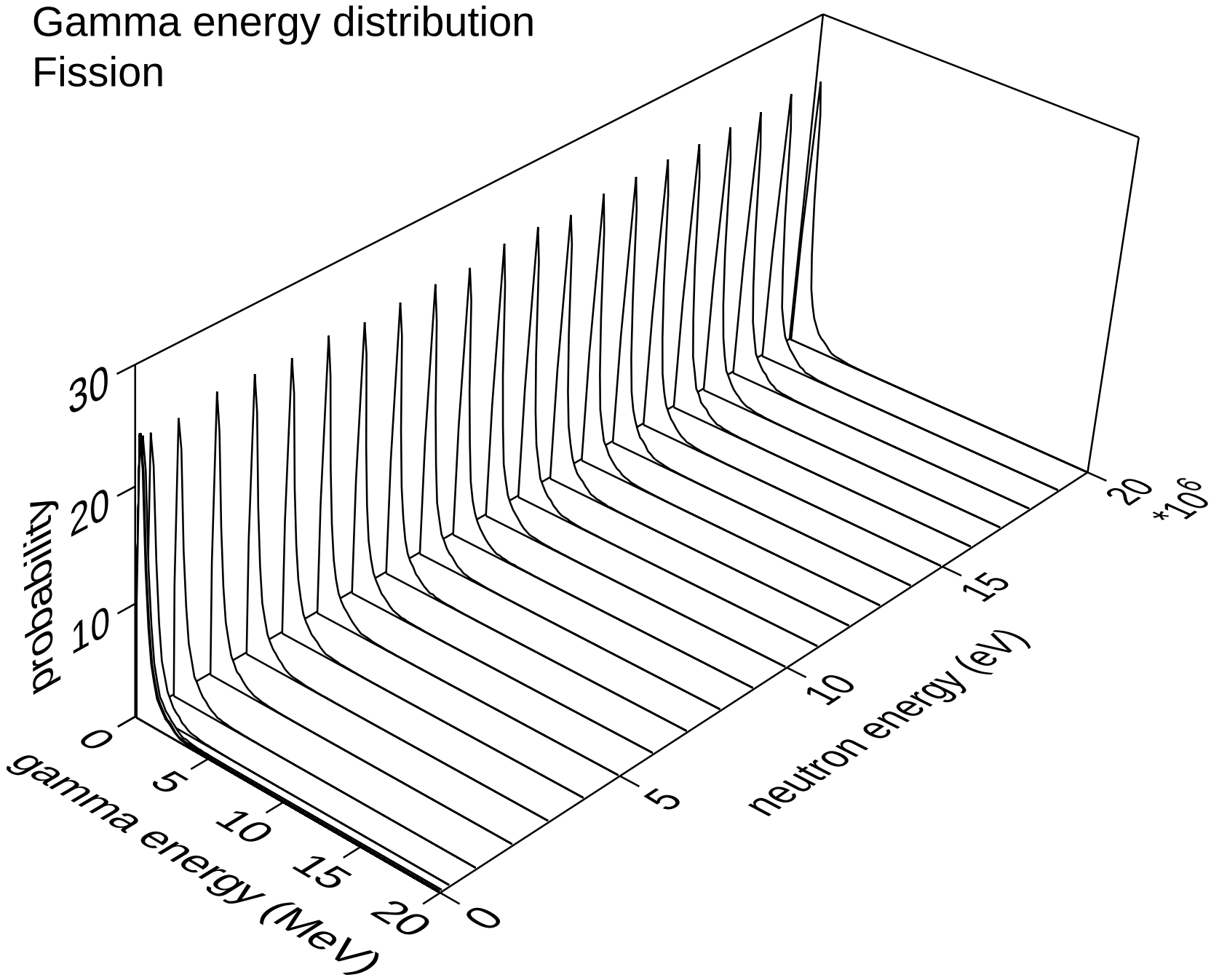

Energy distribution (CMS)
n_n_1 part.=neutron
probability
10^{-1}
0
5
10
15
20
*10^6
energy (eV)
0
5
10
15
20
*10^6
neutron energy (eV)

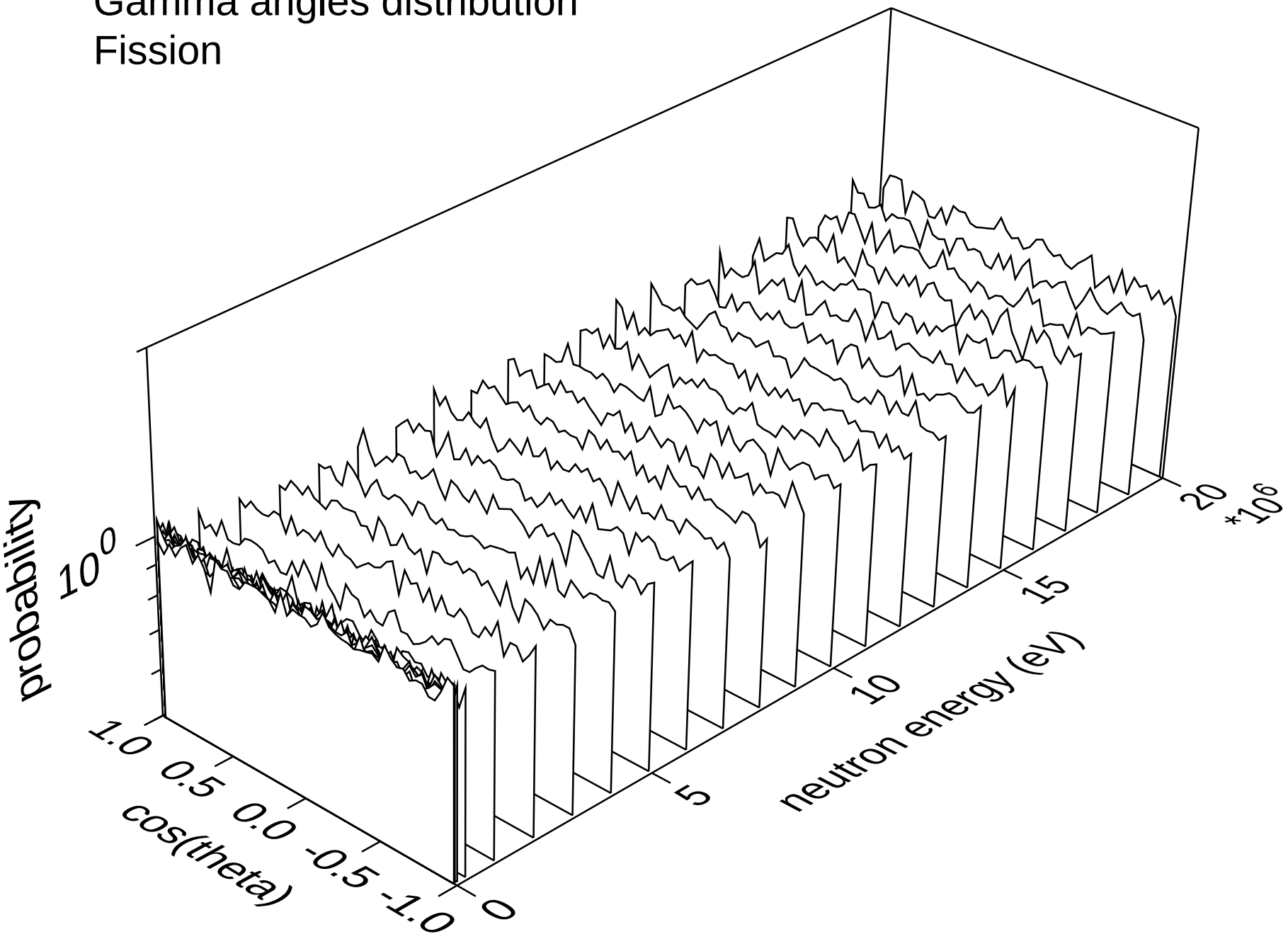
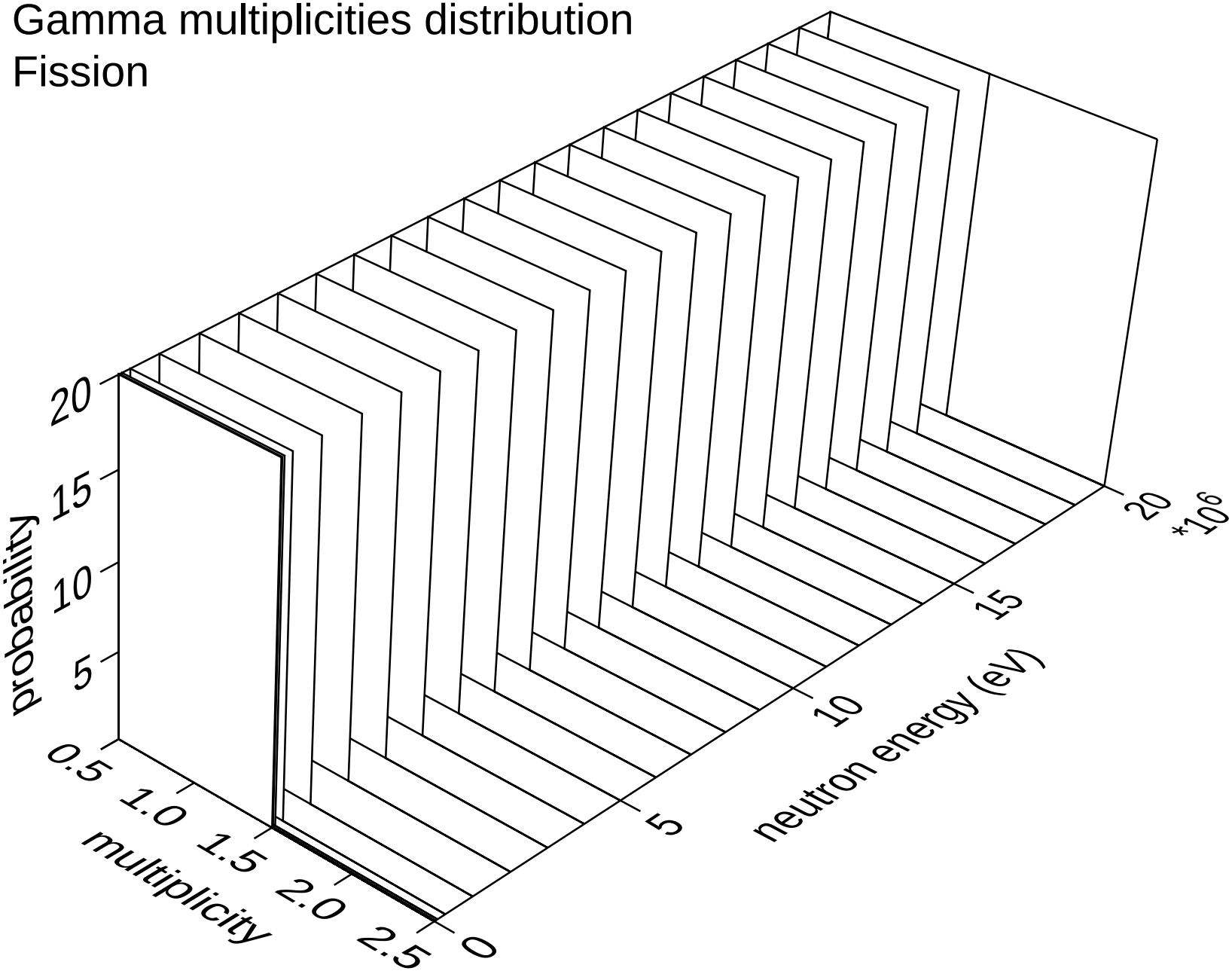

Energy distribution (CMS)
n_n_1 part.=gamma
probability
10^2
10^1
0
200
400
600
800
1000
energy (eV) *10^3
0
5
10
15
20
neutron energy (eV) *10^6


Energy distribution (CMS)
n_n_cont part.=neutron
probability
10^{1}
10^{-1}
0
5
10
15
20
*10^6
energy (eV)
0
5
10
15
20
*10^6
neutron energy (eV)


Energy distribution (CMS)
n_n_cont part.=gamma
probability
10^2
10^1
0
200
400
600
800
1000
0
energy (eV) *10^3
5
10
15
20
*10^6
neutron energy (eV)


Energy distribution (CMS)
Fission prompt
probability
10^{1}
10^{-1}
0
5
10
15
20
*10^6
energy (eV)
0
5
10
15
20
*10^6
neutron energy (eV)


Gamma energy distribution
Fission
probability
0
10
20
30
gamma energy (MeV)
0
5
10
15
20
neutron energy (eV)
5
10
15
20
*10^6


Gamma angles distribution
Fission
probability
10^0
1.0
0.5
0.0
-0.5
-1.0
cos(theta)
0
5
10
15
20
*10^6
neutron energy (eV)

Gamma multiplicities distribution
Fission